## Fusion Reactor Technology I (459.760, 3 Credits)

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### Contents

Week 1. Magnetic Confinement Week 2. Fusion Reactor Energetics (Harms 2, 7.1-7.5) Week 3. Tokamak Operation (I): Basic Tokamak Plasma Parameters (Wood 1.2, 1.3) Week 4. Tokamak Operation (II): Startup Week 5. Tokamak Operation (III): Tokamak Operation Mode Week 7-8. Tokamak Operation Limits (I): Plasma Instabilities (Kadomtsev 6, 7, Wood 6) Week 9-10. Tokamak Operation Limits (II): Plasma Transport (Kadomtsev 8, 9, Wood 3, 4) Week 11. Heating and Current Drive (Kadomtsev 10) Week 12. Divertor and Plasma-Wall Interaction Week 13-14. How to Build a Tokamak (Dendy 17 by T. N. Todd)

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Week 1. Magnetic Confinement Week 2. Fusion Reactor Energetics (Harms 2, 7.1-7.5) Week 3. Tokamak Operation (I): Basic Tokamak Plasma Parameters (Wood 1.2, 1.3) Week 4. Tokamak Operation (II): Startup Week 5. Tokamak Operation (III): Tokamak Operation Mode Week 7-8. Tokamak Operation Limits (I): Plasma Instabilities (Kadomtsev 6, 7, Wood 6) Week 9-10. Tokamak Operation Limits (II): Plasma Transport (Kadomtsev 8, 9, Wood 3, 4) Week 11. Heating and Current Drive (Kadomtsev 10) Week 12. Divertor and Plasma-Wall Interaction Week 13-14. How to Build a Tokamak (Dendy 17 by T. N. Todd)

# • MARFE (Multi-faceted Asymmetric Radiation From the Edge)

#### MARFE: AN EDGE PLASMA PHENOMENON

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ABSTRACT. A tokamak edge phenomenon, dubbed the 'marfe' (for multifaceted asymmetric radiation from the edge), is described. This phenomenon, observed in medium- to high-density Alcator C discharges, is characterized by greatly increased radiation, density and density fluctuations, and decreased temperature in a relatively small volume at the inner major radius edge of the plasma. The marfe appears to be confined to minor radii greater than or of the order of that of the limiter. The affected region is typically above the midplane, extending poloidally for about 30° and toroidally for  $360^{\circ}$ . The temperature and density of the core plasma are unaffected by the marfe. A simple transport model is used to show that the marfe is the manifestation of a thermal instability, with impurity radiation being the main energy loss mechanism out of the marfe volume. A density threshold  $n_m$  for marfe onset is observed;  $n_m$  is found to be an increasing function of plasma current and a decreasing function of intrinsic low-Z impurity levels. Detailed observations from spectroscopy, bolometry, Langmuir probe measurements, interferometry and CO<sub>2</sub> scattering are presented.

B. Lipschultz et al, NF 24 977 (1984)





Shot number : 4207 2010/11/08	001	0:00:00:00	Shot number : 3876 2010/10/26	001	0:00:00:00
KSTAR TV1 (t=-100ms)			KSTAR TV1 (t=-100ms)		

Shots selected by Dr. Y. M. Jeon (NFRI) 6



#### MARFE (Multi-faceted Asymmetric Radiation From the Edge)



M. Greenwald, "Density Limits in Toroidal Plasmas", APS (2001)

#### • MARFE

- First observed in medium- to high-density in ALCATOR-C discharges J. L. Terry et al, Bull. Am. Phys. Soc. **26** 886 (1981)
- Characterised by a toroidal ring of a dense moderately cold plasma, located at the periphery of a plasma column on its inner contour
- Edge impurity radiation is both in/out and up/down asymmetric, before and during a MARFE.
- Relatively small MARFE region emits a large fraction of the total radiated power.
- Easily observed due to its intense light radiation:
   High plasma density: ion density increased by a factor of up to ten comparable with the central density of the main plasma
   Low temperature: temperature dropped by 50% or so several eVs

#### • MARFE

 Thermal-radiation instability observed in the at high densities near the density limit: Temperature decreases (due to radiation of → radiation increases

$$P_{line} = n_{19} \overline{n}_{19}^* A / \hat{T}_e^{\alpha} \quad (\alpha > 0)$$

density of radiating elements



The emissivity of most of the important impurities (mainly carbon from the wall materials) reach maxima at temperature in the range of 10 - 200 eV.

- $\rightarrow$  temperature continuously decreases
- → plasma pressure along the magnetic field increases plasma density
- $\rightarrow$  radiation further enhanced
- $\rightarrow$  a region of cold plasma (MARFE) formed
- $\rightarrow$  (sometimes) L-mode disruption



#### • MARFE

- Edge plasma 'compresses' MARFE cold plasma (plasma flows into the MARFE, increasing the density) along magnetic field lines to maintain pressure balance and feeds the energy for the subsequent re-radiation: radiative condensation
- MARFE forms on closed flux surfaces inside the main plasma on a poloidal location where the temperature has a minimum: in a cylindrical limiter tokamak at the high-field side near the inner wall and in a divertor tokamak near the X-point.
- Outcome: not always the loss of H-mode confinement

#### MARFE: measurements

- Discovered by the observation of an increased localized impurity emission from the MARFE edge by bolometry and visible spectroscopy
- Also detected by the bremsstrahlung from the high-density core of the MARFE in ASDEX Upgrade.
  - The temperature in the MAFRE centre can drop below 1 eV so that the plasma recombines by three-body recombination.
  - The three-body recombination in MARFE was detected by the characteristic Balmer spectrum near the series limit.



U. Wenzel et al, Plasma Phys. Control. Fusion 44 L57 (2002)

#### Snakes

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**16 NOVEMBER 1987** 

#### Persistent Density Perturbations at Rational-q Surfaces Following Pellet Injection in the Joint European Torus

A. Weller, <sup>(a)</sup> A. D. Cheetham, A. W. Edwards, R. D. Gill, A. Gondhalekar, R. S. Granetz, <sup>(b)</sup> J. Snipes, and J. A. Wesson JET Joint Undertaking, Abingdon, Oxon OX143EA, United Kingdom

(Received 9 February 1987)

In the Joint European Torus the ablation of injected pellets produces a striking resonance effect when the pellets reach surfaces with q values 1 and  $\frac{3}{2}$ . Subsequently, structures with mode numbers m=1, n=1 and m=3, n=2 are observed with the soft-x-ray cameras for up to 2 s as compact snakelike perturbations. These structures, which persist through several sawtooth collapses, give information on the radii of the q=1 and  $q=\frac{3}{2}$  surfaces and the q-profile evolution. The observations can be explained by the formation of magnetic islands.

PACS numbers: 52.55.Fa, 52.30.-q

#### • Snakes



#### Snakes

- First observed in JET: a rope-like filament observed in the soft-X-ray emission following the injection of a D<sub>2</sub> pellet
- A relatively cool, high density structure with typical poloidal and radial dimensions of  $I_{\theta} \sim 25$  cm and  $I_r \sim 17$  cm that forms on the q = 1 surface and which rotates about the minor axis
- While q = 1 is the preferred value, similar structures can occur on the q = 3/2 surface.
- Surviving for ~ 2 s regardless of frequent disturbances from sawtooth oscillations
- Pellet penetration needs to be inside the q = 1 surface to form the snakes.

#### Snakes

- Density and temperature of a typical snake:

 $\Delta n = 3 \times 10^{19} \text{ m}^{-3}, n_b = 6 \times 10^{19} \text{ m}^{-3}, \Delta T_e = -140 \text{ eV}, T_b = 1200 \text{ eV}$ (b : background values)

total number of particles in the snake  $\sim$  1% of the pellet particles

- Following an initial large drop attributed to the energy required to ionize the pellet atoms,  $T_e$  within the snake quickly rises to within 10% or so of the ambient plasma temperature.



#### Snakes

- Cool deuterium atoms supplied are swept outwards by the radial plasma motion until reaching a cool channel, C of the q = 1 surface (bottom of the sawtooth oscillation), where if the collapse phase of the sawtooth oscillation has just occurred.
- Ionization of the deuterium atoms as they cross C absorbs considerable energy and results in the large temperature drop.
   Equilibrium will require a nearly constant pressure and therefore initially, when C is relatively cool, it will become appreciable denser than its surroundings.
- Further progress to a fully developed snake depends on the transport of more particles into C and the maintenance of a temperature depression.

#### Snakes: application to diagnostics

- Acting as a sort of probe for studying of the position of the q = 1 surface during a sawtooth cycle
- Angular velocity of the snake about the minor axis providing a diagnostic of ion temperature, radial electric field  $(E_r)$

$$E_r = v_{\phi} B_{\theta} - v_{\theta} B_{\phi} + p_i' / e n_e$$

#### • Disruptions





#### Disruptions

- Disruptions are fast (~1 ms) global instabilities that my arise in magnetic confinement fusion devices that use plasma current for confinement such as tokamak, ST, etc.
- Termination of confinement, uncontrolled loss of thermal and magnetic energy
  - shift of the plasma column
  - heat load damage to plasma facing components (PFCs)
  - large mechanical stresses from JxB forces during current quench (large negative voltage spike in the transformer)
  - rapid cooling of the plasma
  - Highly efficient conversion of poloidal magnetic energy into "runaway" electrons through avalanche amplification, resulting in a > 5 MA of relativistic electron beam

#### • Disruptions

- Several classes of "triggering" instabilities lead to this "final" ideal instability

- Beta / pressure limits
- Radiative limits
- Vertical position instability

(Vertical Displacement Event (VDE))

- Disruptions in KSTAR VDE (#2265) low-q (#2271-3) density limit (#2277, #5321)



#### Disruptions



- Thermal quench: Rapid loss of plasma thermal energy, global MHD activity

Current quench:
Resistive current decays due to
lowered plasma temperature,
loss of confining poloidal field

#### • Disruptions

- By necessity, burning plasmas for fusion energy production will have high thermal and magnetic energy densities, making the problem of disruption damage much more severe than on present confinement experiments.
- End-of-lifetime damage will occur to internal components of burning plasma devices as a result of the uncontrolled loss of thermal and magnetic energy associated with disruptions.
- 1. Heat load damage to plasma-facing surfaces.
- 2. Large mechanical stresses from **J**x**B** forces during current quench.
- 3. Highly efficient conversion of poloidal magnetic energy into "runaway" electrons ( $E_{\phi}$  induced to sustain poloidal magentic flux) through avalanche amplification if  $E_{\phi} > E_{Dreiser}$  (Rosenbluth et al.), resulting in a > 5 MA of relativistic electron beam.
- As a consequence, disruptions drive up cost and decrease flexibility for design choices of next-step burning plasma experiments.

#### Disruptions

- Halo current:

Induced to sustain magnetic flux Can reach ~50% of plasma current before disruption EM force induced by Halo Current x  ${\bf B_t}$ 



#### Disruptions

- Divertor target thermal loading illustrates the severity of disruption damage for burning plasma devices.
- Extrapolate damage threshold in poloidal divertor tokamak:
  - Stored thermal energy  $W_{th}$  (~3*nkTV*) is "lost" in MHD/conduction time scale  $t_{TO}$
  - Divertor impulse heating onto divertor wetted area, A<sub>divertor</sub>

 $W_{th}$  /  $A_{divertor}$  /  $t_{TQ}^{1/2} \rightarrow \Delta T_{divertor \ plates}$ 

Ex) Expected heat loads in thermal quench time  $\sim 0.5$  ms:

Device	<i>W<sub>th</sub></i> (МЈ)	A <sub>divertor</sub> (m <sup>2</sup> )	<b>∆T</b> figure of merit (MJm <sup>-2</sup> s <sup>-1/2</sup> )
DIII-D	~ 1	2	~25
ITER	~ 300	25	~550

- Ablation limit of carbon / melt limit of tungsten: ~50  $MJm^{-2}s^{-1/2}$ 

- The power densities associated with a burning plasma disruption will easily surpass damage threshold for any divertor design.

#### Disruptions

- Burning plasma experiments must develop a thorough strategy to deal with disruptions.
- 1. Plasma operations:
  - Obtain needed performance away from known stability limits
  - (e.g. keep plasma current and *n* within stable limits, close fitting conducting wall)
- 2. Disruption avoidance:
  - Control of plasma pressure / current profiles
  - (e.g. (2,1) NTM suppression)
- 3. Disruption detection:
  - Reliably determine onset of triggering instability in real-time
  - (e.g. neural network)
- 4. Disruption mitigation:
  - Provide a rapid and safe emergency shutdown technique in order to alleviate damage to costly internal components

#### Disruption Mitigation

- Killer pellet injection: fast conversion of thermal energy to the radiation energy
- MGI (Massive Gas Injection): H, He, Ne, Ar, Kr, Xe, etc.
- RMP to reduce runaway electrons



Neon gas jet injection triggered by control system

#### Disruption Mitigation

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# Disruption



#### References

- D. G. Whyte, "The Consequences of Disruptions for Burning Plasma Experiments", Plasma Physics Colloquium, Columbia University, 17 October 2003