

# **Fusion Reactor Technology II**

**(459.761, 3 Credits)**

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Week 9. Radioactivation

Week 10. Blanket Structure and Breeding Materials

Week 11-12. Types of Blanket in ITER and DEMO

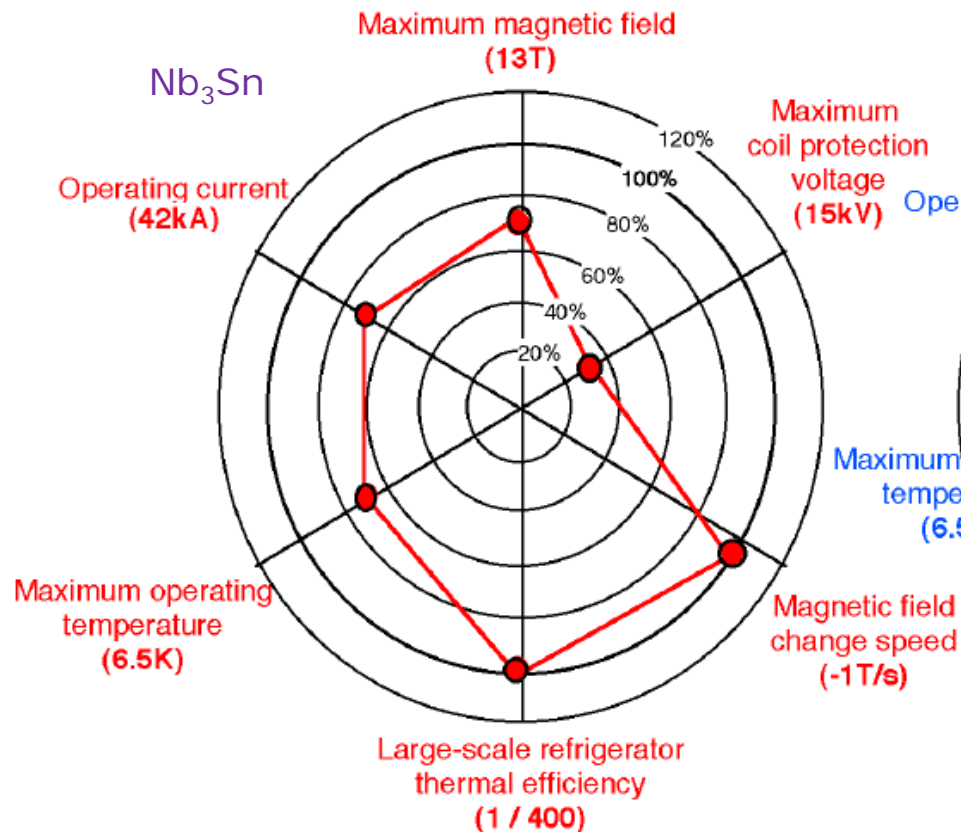
Week 13. Plasma Facing Components

Week 14. Fuel Cycle System

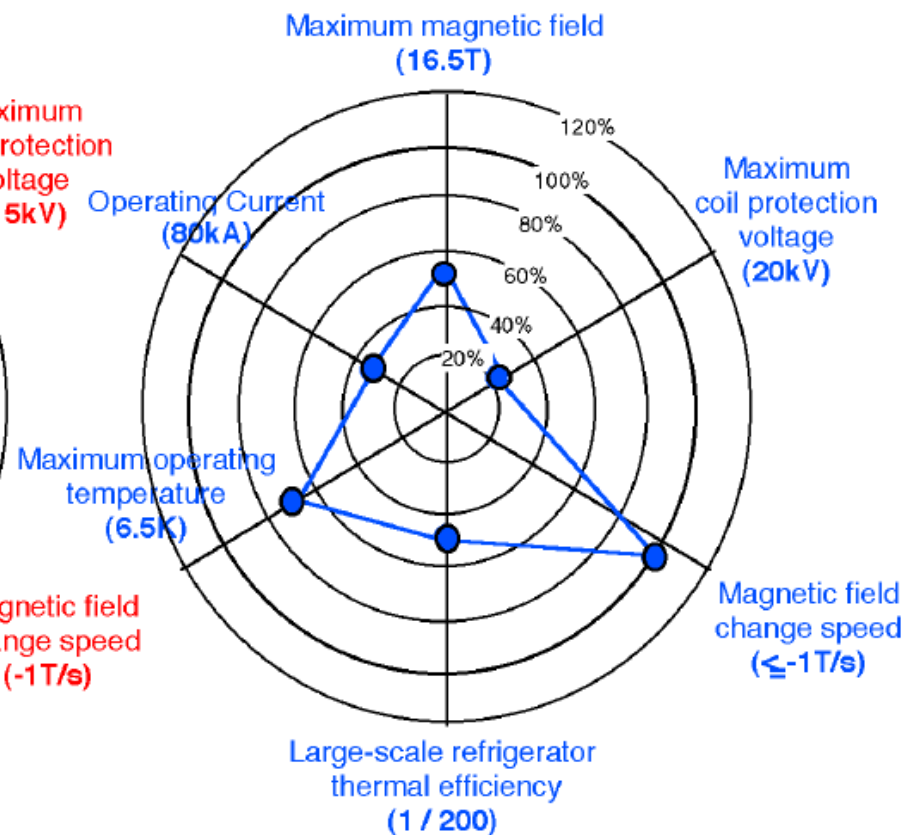
Fusion Plasma Technology  
**Reactor Technology**  
Blanket and Material Technology  
Safety Technology  
Operation and Maintenance  
Technology

# Superconducting magnet technology

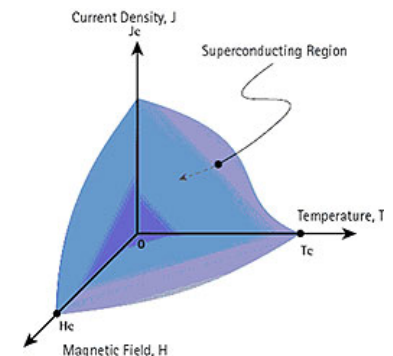
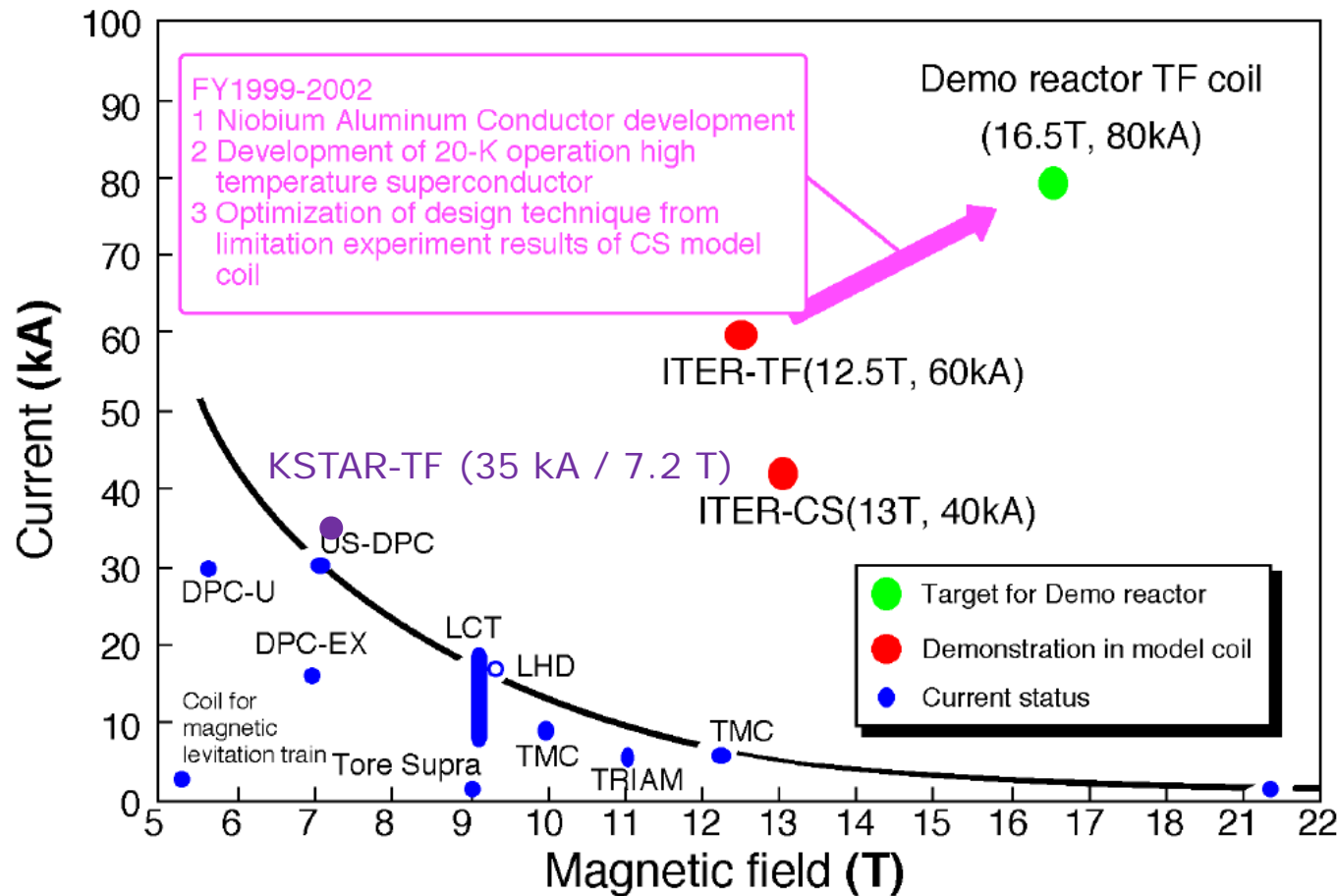
For ITER



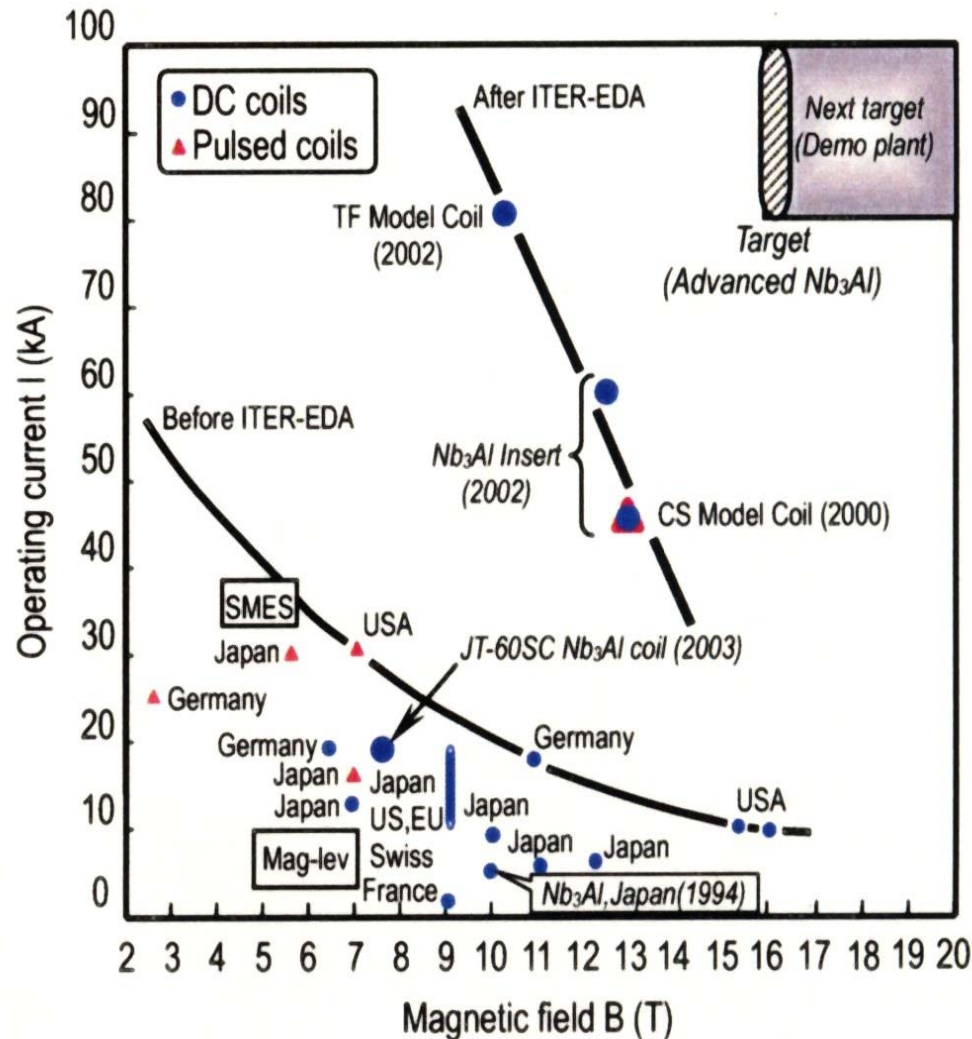
For Demo Reactor



# Superconducting magnet technology

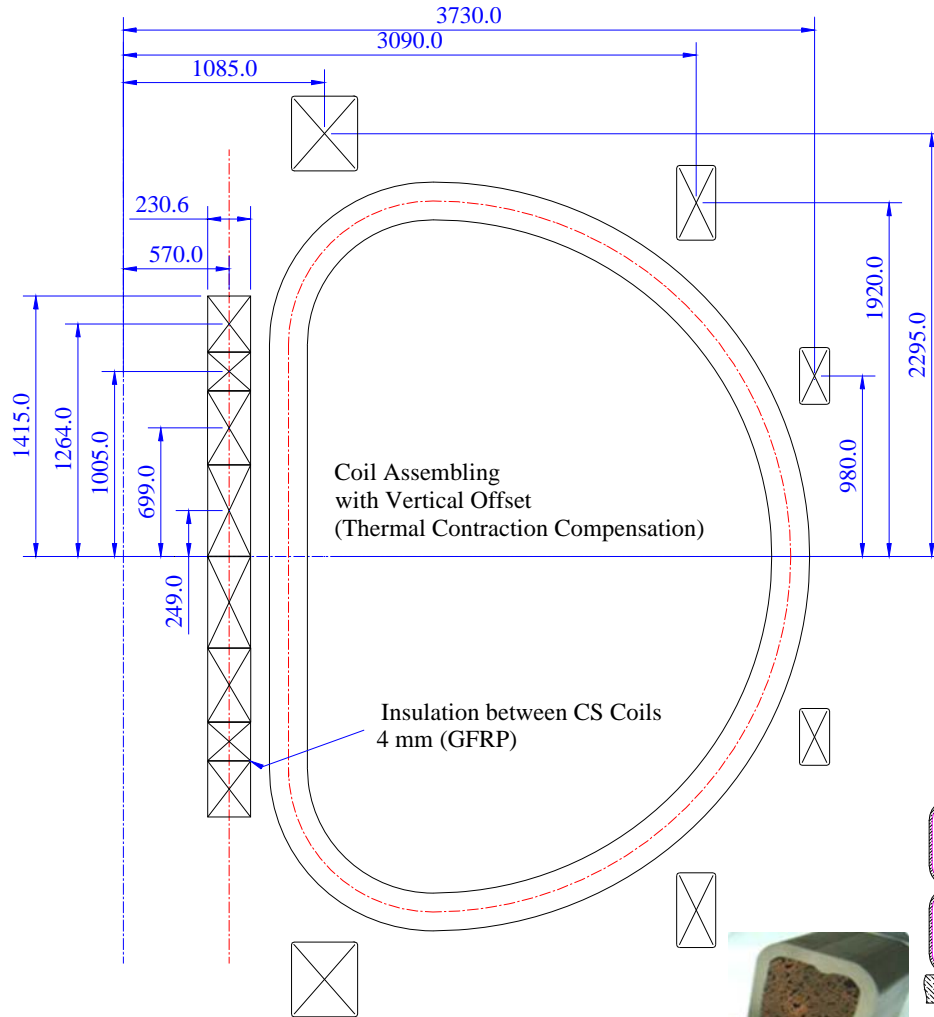


# Superconducting magnet technology



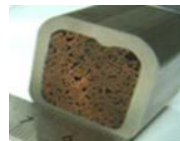
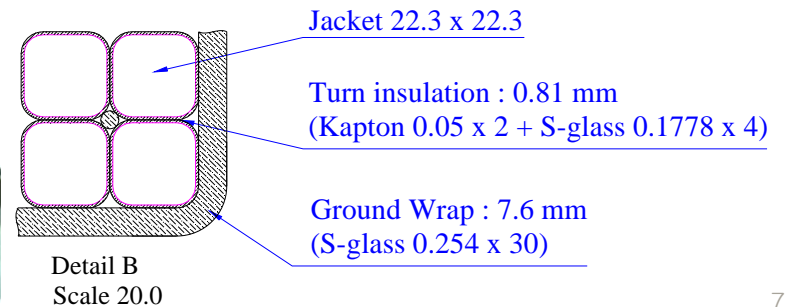
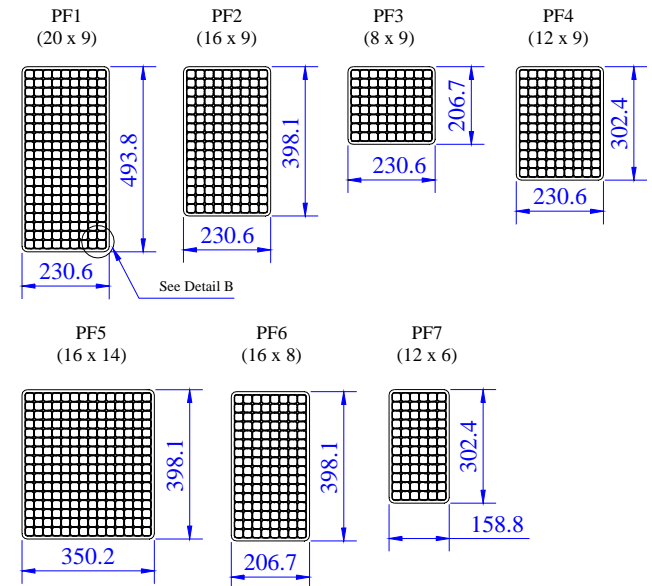
*Operating points of superconducting coils constructed so far and the target for fusion demo plant [N. Koizumi, et al., 20<sup>th</sup> IAEA Fusion Energy Conf. IAEA-CN116-FT/P1-7]*

# Layout of the KSTAR magnet



Insulation Thickness :  
Turn insulation : 0.81 mm  
Ground wrap : 7.6 mm

Coil Cross Section (Scale 2.0)





# Superconducting magnet technology

## ITER conductors in 2007

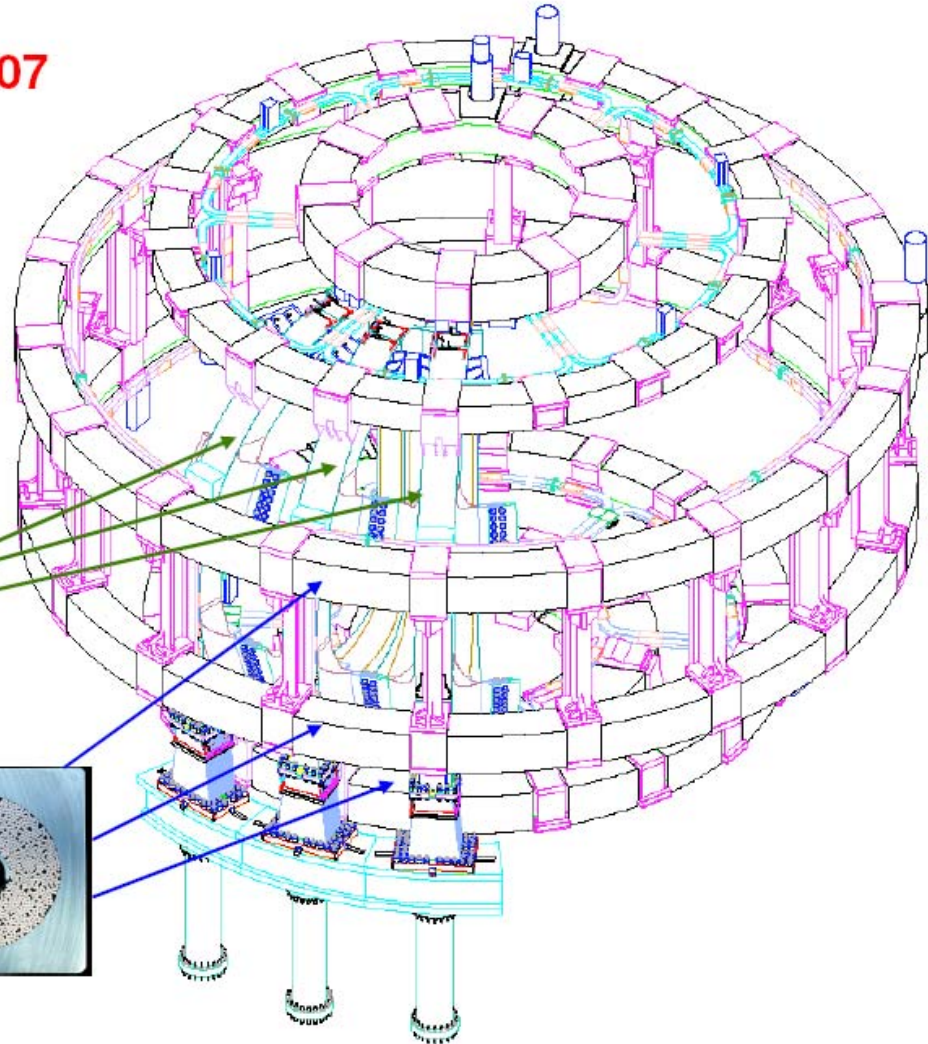
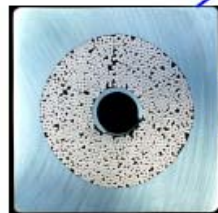
CS Conductor,  
 $\text{Nb}_3\text{Sn}$



TF Conductor,  
 $\text{Nb}_3\text{Sn}$



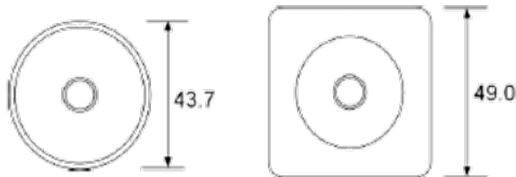
PF Conductor,  
 $\text{NbTi}$





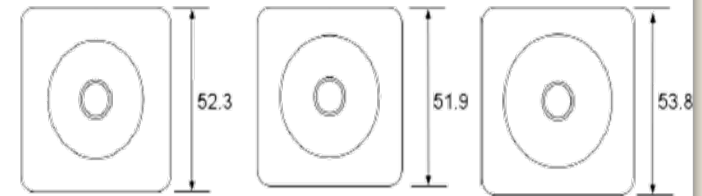
# ITER conductor

## Nb<sub>3</sub>Sn conductor



	TF	CS
Iop (kA)	68	40 (IM) 45 (EOB)
Bmin – Bmax (T)	10.5 – 11.8	12.4 - 13.0 (IM) 12.0 - 12.6 (EOB)
Top (K)	5.0	4.7
ε (%)	-0.77	-0.69
τ discharge (s)	11 + 2s delay	7.5 + 2s delay
sc strand diam. (mm)	0.82	0.83
sc strand Cu:nonCu	1	1
cabling layout	((2sc+1Cu) x 3 x 5 x 5 +core) x 6	(2sc+1Cu) x 3 x 4 x 4 x 6
core in 4 <sup>th</sup> stage	3 x 4 Cu wires 0.82 mm	na
Cu strand in 1st triplet	1	1
sc strand Nr	900	576
local Vf (%)	33.2	33.2
cable diam. (mm)	40.5	32.6
central spiral od x id (mm)	9 x 7	9 x 7
flow area in annulus (mm <sup>2</sup> )	406.5	252.3
total flow area (mm <sup>2</sup> )	445.0	290.8

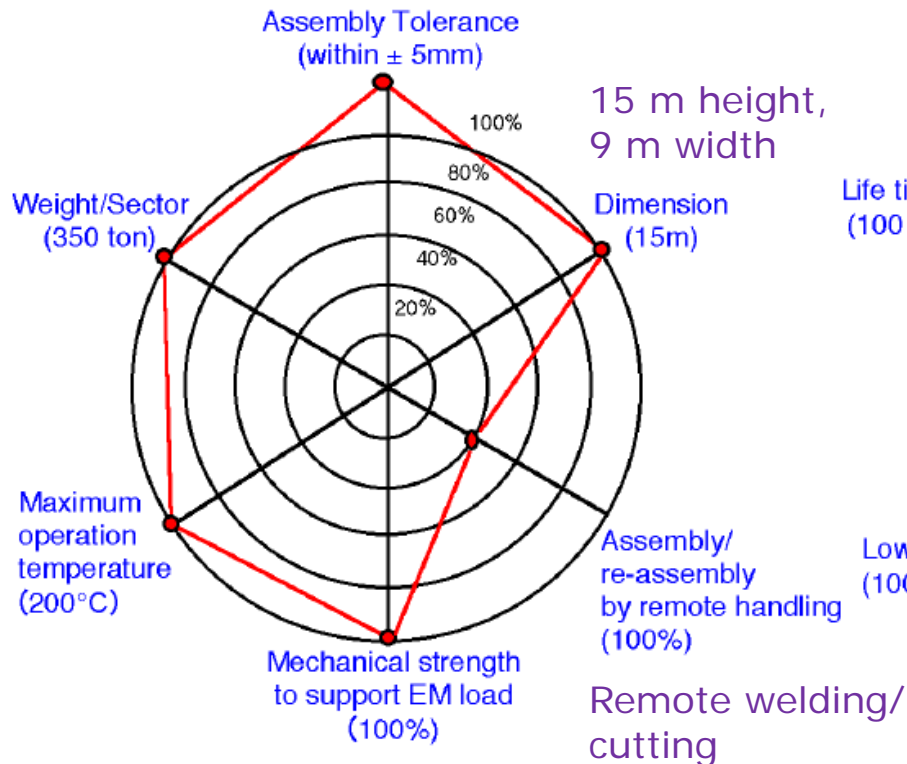
## PF conductor



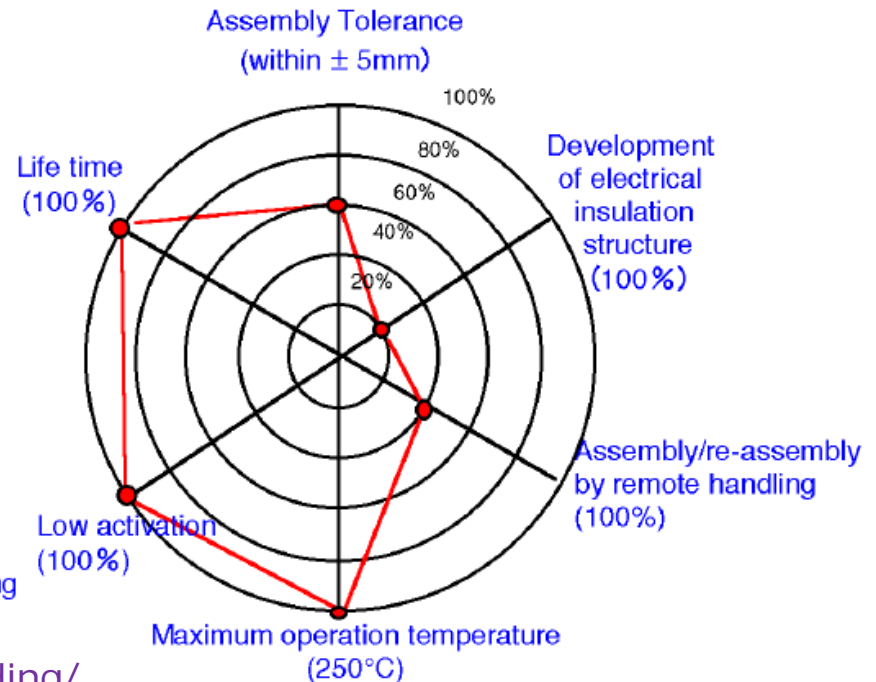
	PF2/3/4	PF5	PF1/6
Iop (kA)	45	45	45
Bpeak (T)	4	5	6
Top (K)	5.0	5.0	5.0
τ discharge (s)	14 +2s delay	14 +2s delay	14 +2s delay
sc strand diam. (mm)	0.73	0.72	0.73
sc strand Cu:nonCu	6.9	4.4	1.6
cabling layout	((3 x 3 x 4+1) x 4+1) x 6	((3 x 3 x 4+1) x 5+1) x 6	3 x 4 x 4 x 5 x 6
Cu core diam 2/3/4 stage (mm)	0.0/1.8/3.5	0/1.2/2.7	0.0/0.0/0.0
sc strand Nr	864	1080	1440
local Vf (%)	34.2	34.3	34.5
cable diam. (mm)	34.5	35.4	38.2
central spiral od x id (mm)	10 x 12	10 x 12	10 x 12

# Vacuum vessel technology

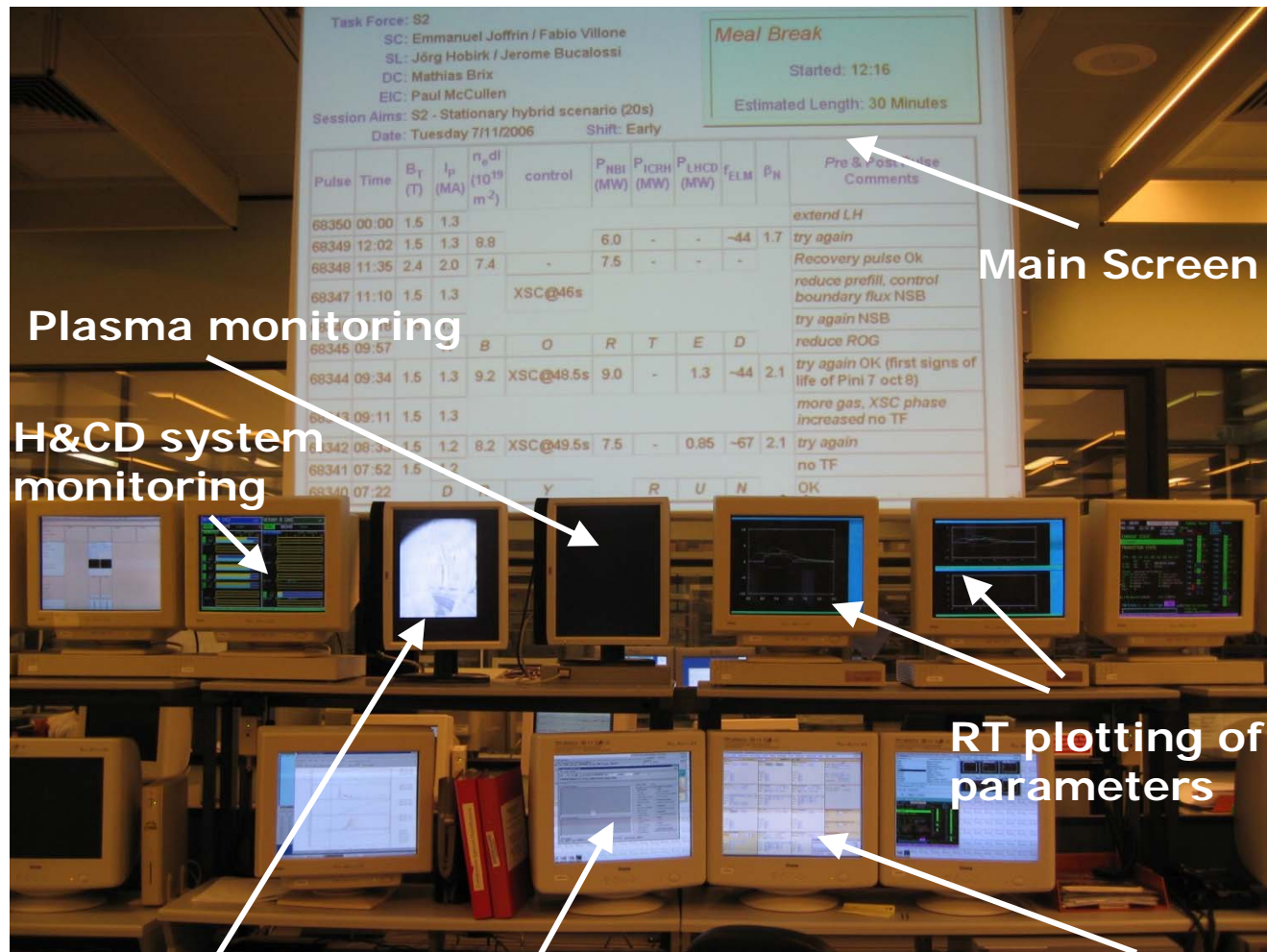
## For ITER



## For Demo Reactor



# JET control room



IR Camera  
(PFC monitoring)

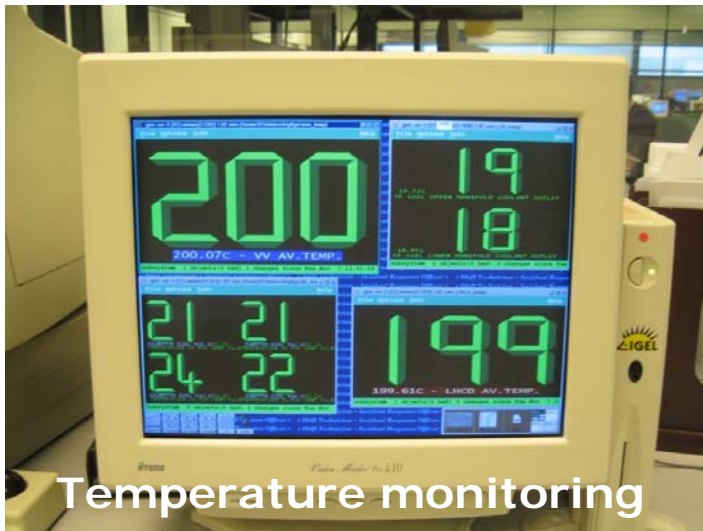
Shot comments

Session Leader

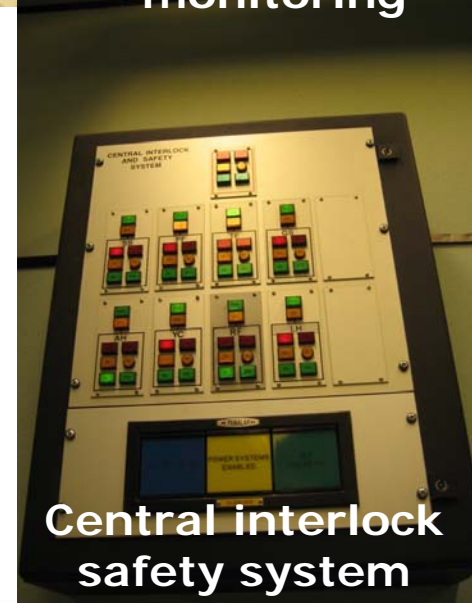
# JET control room



Vacuum system monitoring



Temperature monitoring



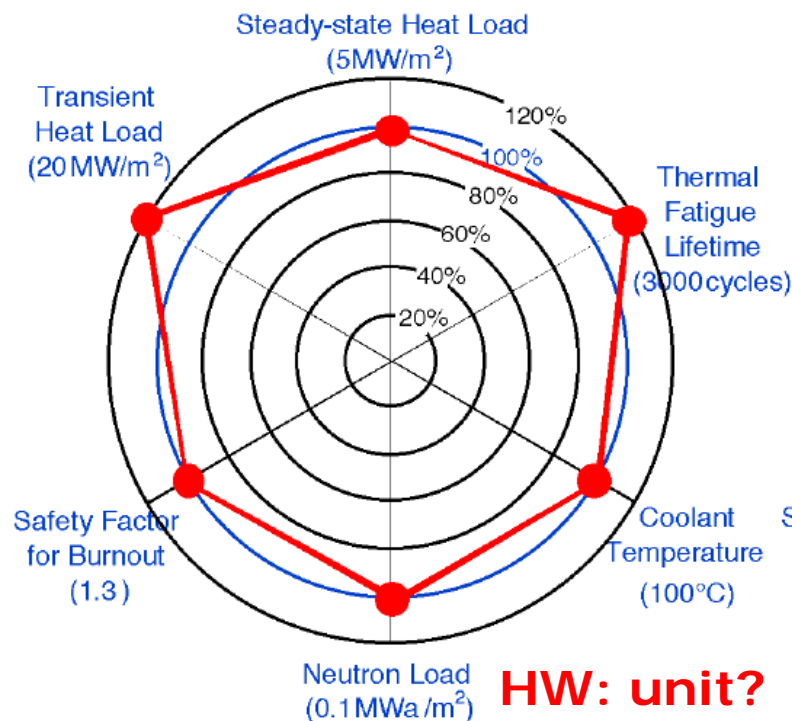
Central interlock safety system



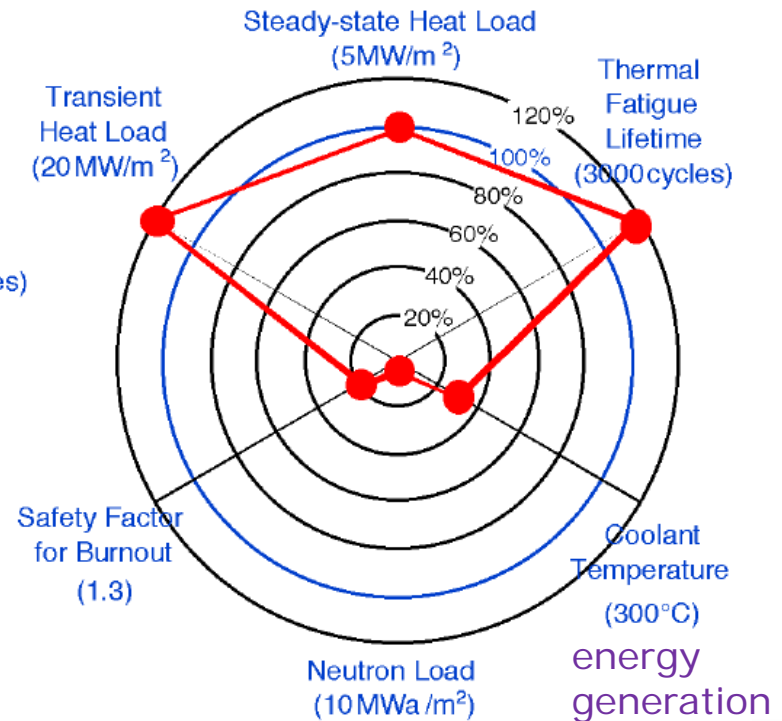
# Divertor and high heat-flux component technology

- ITER divertor requirements
  - heat load: 5-20 MW/m<sup>2</sup>
  - Coolant temperature: 100-150 °C
  - Neutron influence: 0.1 MWa/m<sup>2</sup>

For ITER



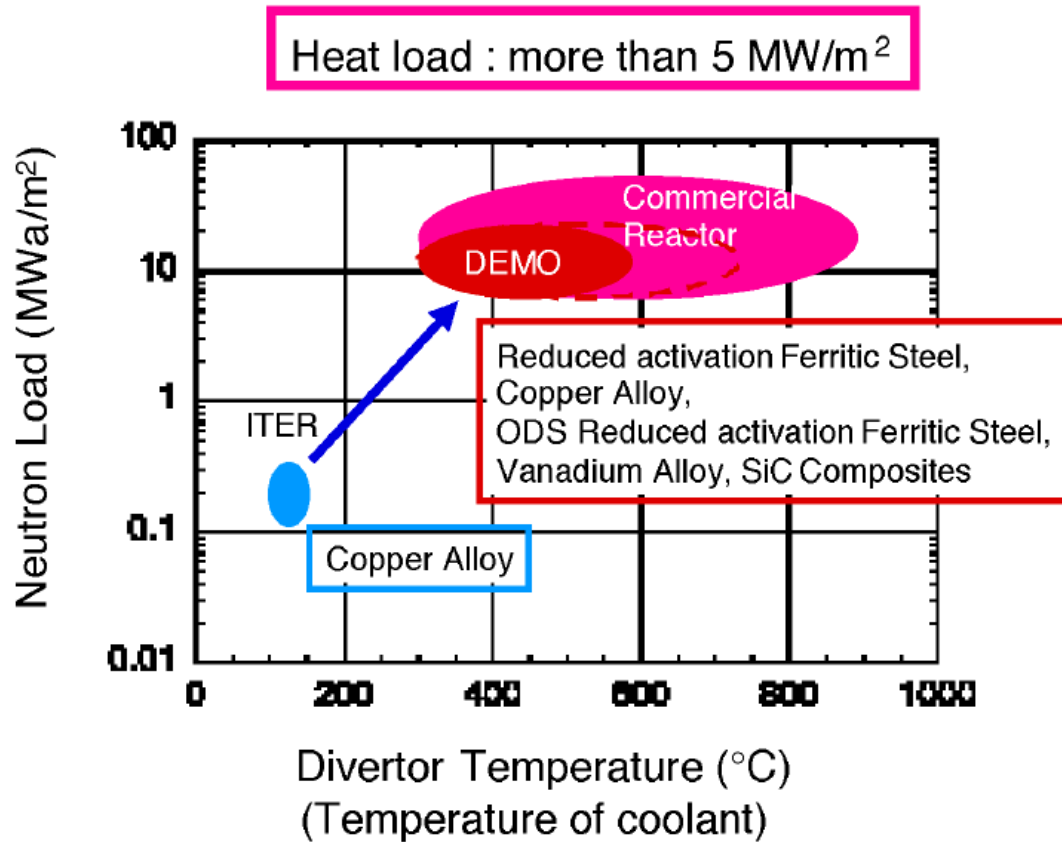
For Demo Reactor



HW: unit?

energy generation

# Divertor and high heat-flux component technology



# Blanket technology

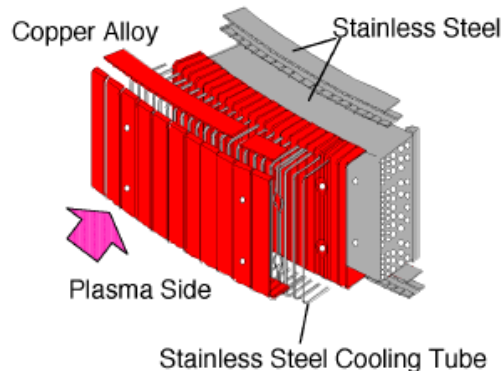
## 1. Shield Blanket for ITER

### ■ Requirements and Structure

- 1) Withstand High Thermal Stress
- 2) Withstand High Electromagnetic Force
- 3) Provide High Shielding Capability



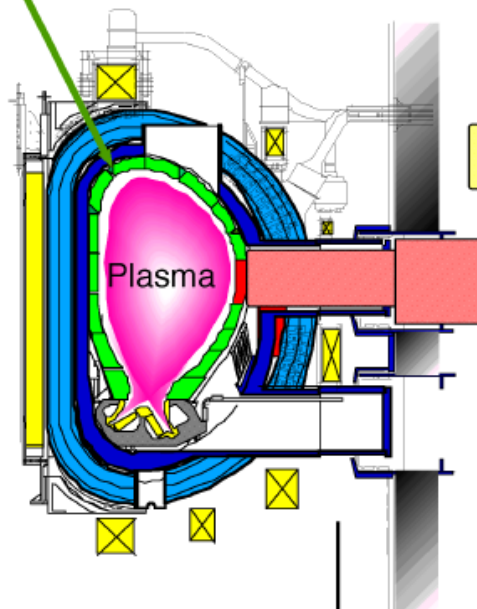
Copper Alloy for Heating Surface  
Stainless Steel for Structure and  
Cooling Tubes



### ■ Technology Challenge

Simultaneous Hot Isostatic Pressing (HIP) bonding of SS/SS, Cu/Cu, and SS/Cu has been successfully developed at 1050°C, 150MPa, and 2 hours holding time, and a prototype blanket module has been completed.

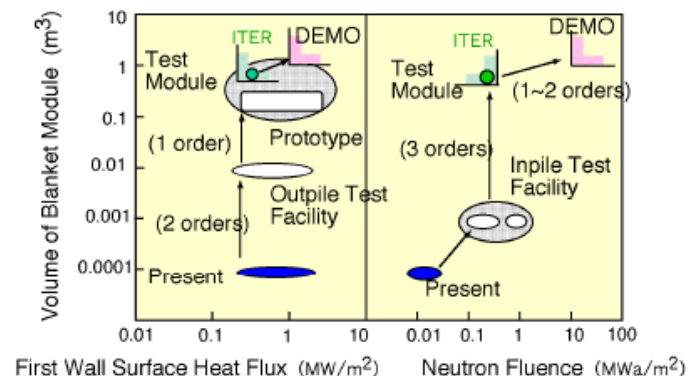
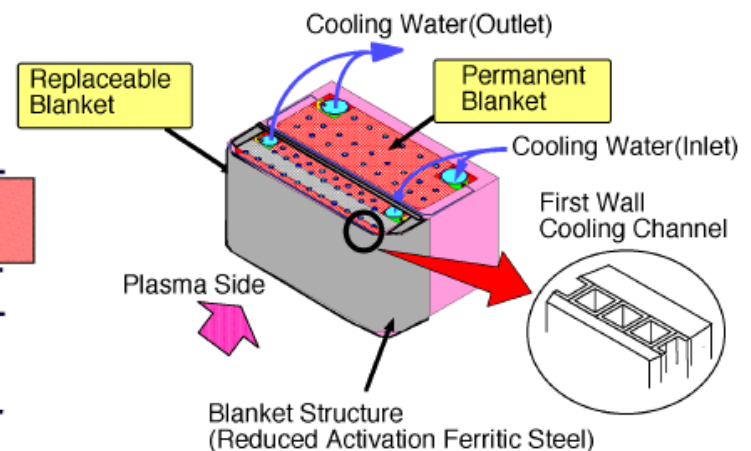
ITER Crosssection



## 2. Blanket for Power Reactor (Tritium Breeding and Power Generation)

Double Layered Structure with Replaceable and Permanent Blankets

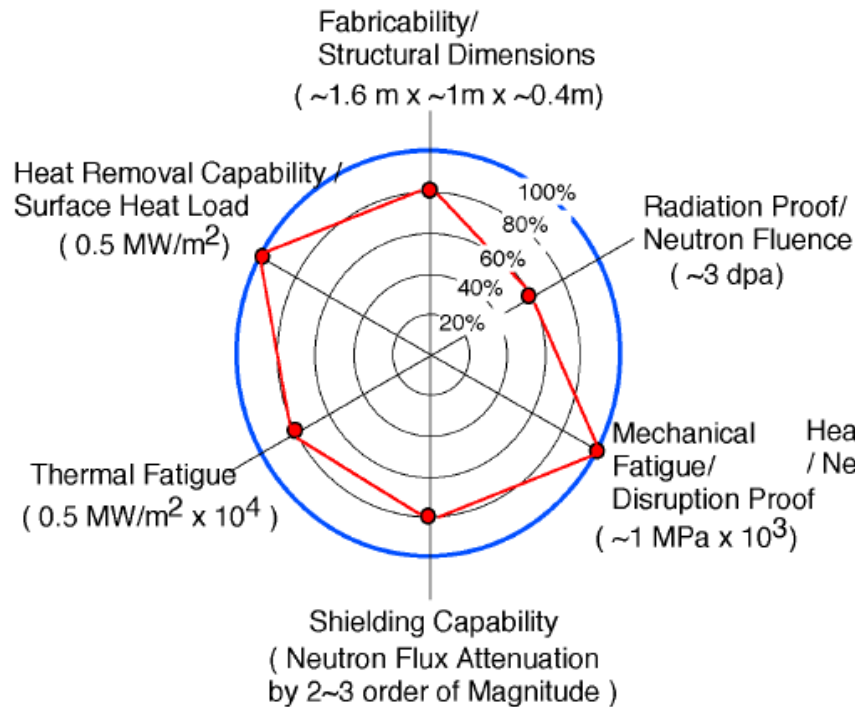
R&D Items: Development of a blanket structure capable of tritium breeding and high heat generation for power generation, under high heat load and neutron wall load





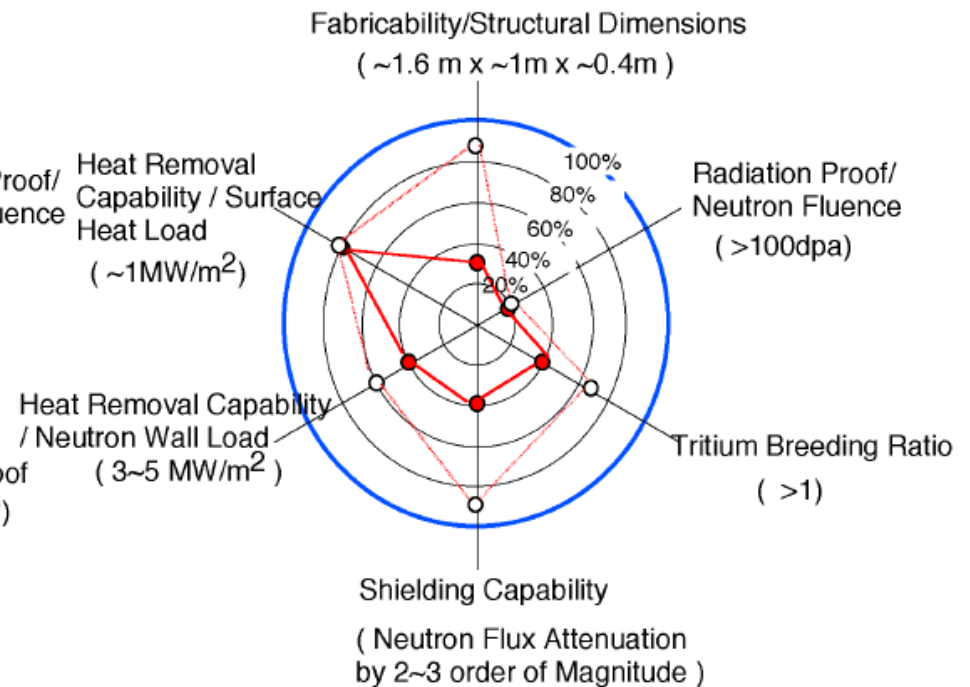
# Blanket technology

## For ITER



## For Demo Reactor

(Reduced Activation Ferritic Steel)



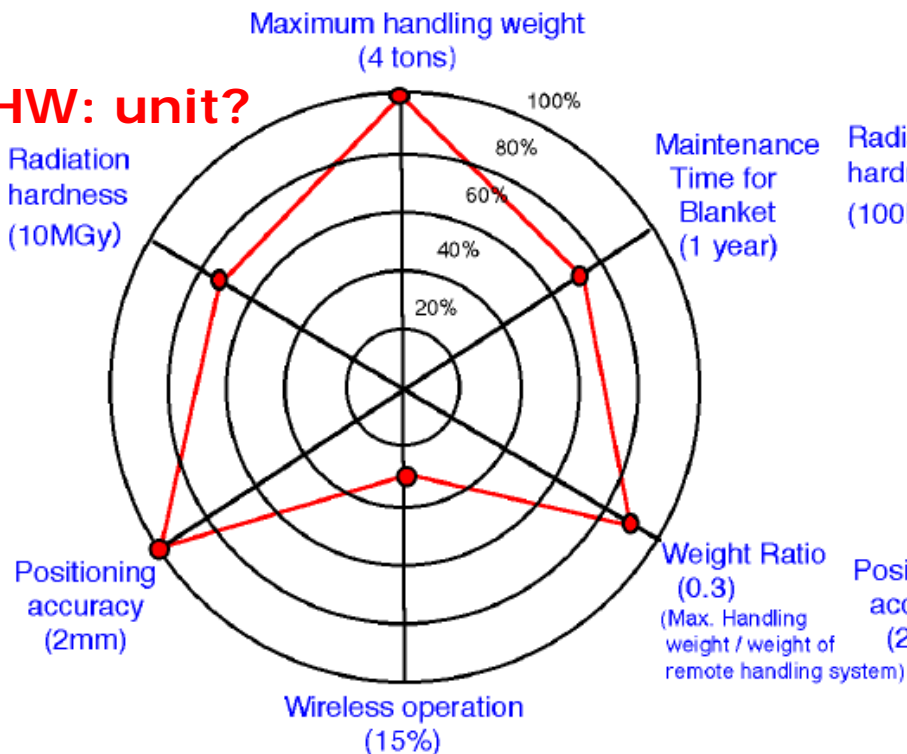
# Remote handling technology

- Minimisation of the maintenance time
- Development of radiation-resistant components  
(radiation-resistant battery, signal transmitter for wireless control)

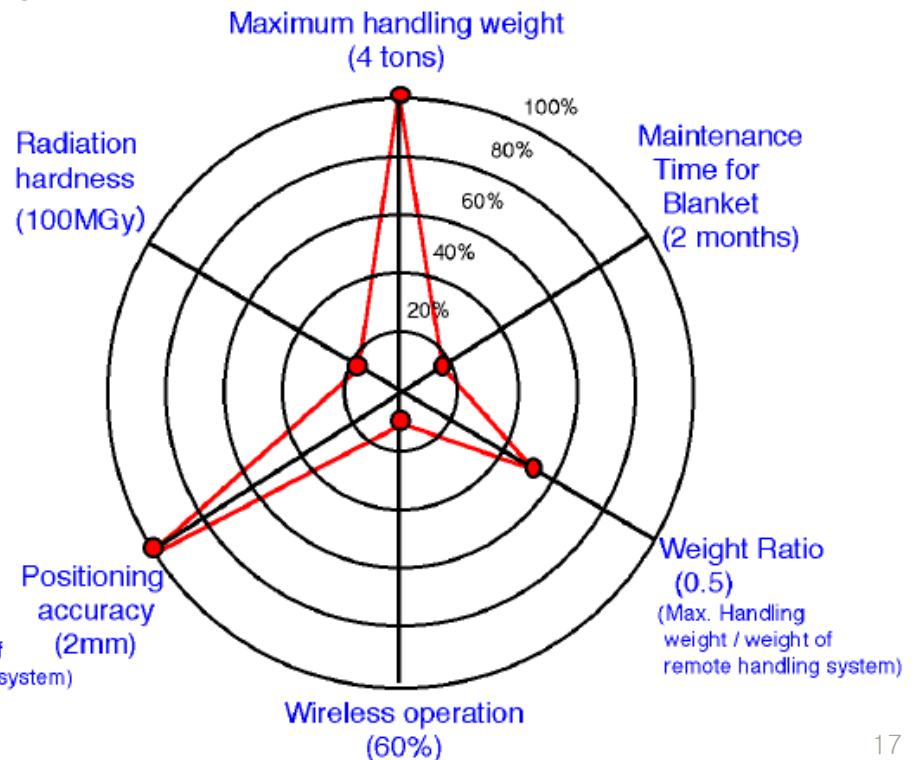
For ITER

4 tons for blanket,  
15 tons for divertors

**HW: unit?**



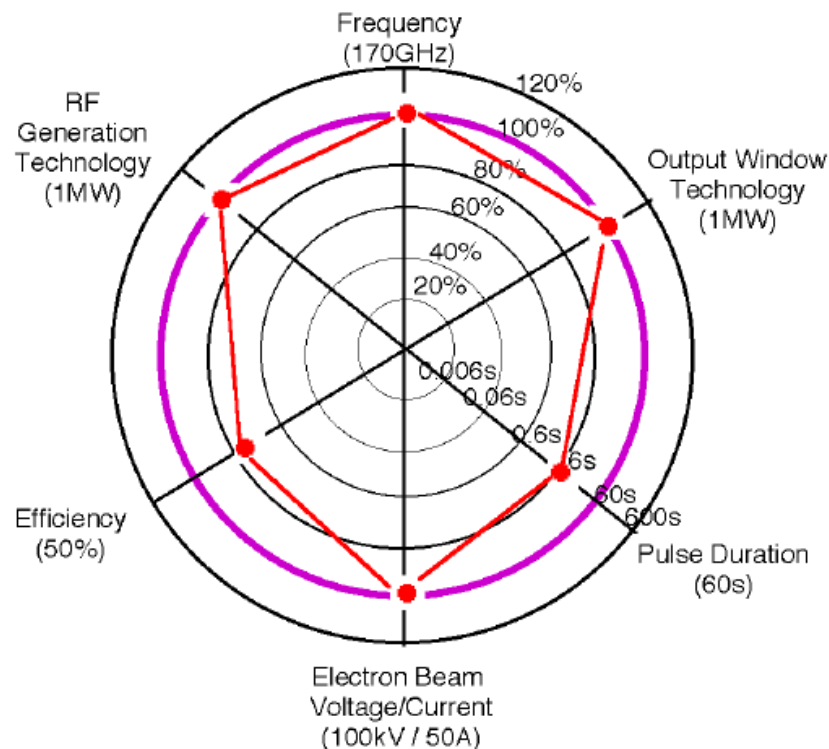
For Demo Reactor



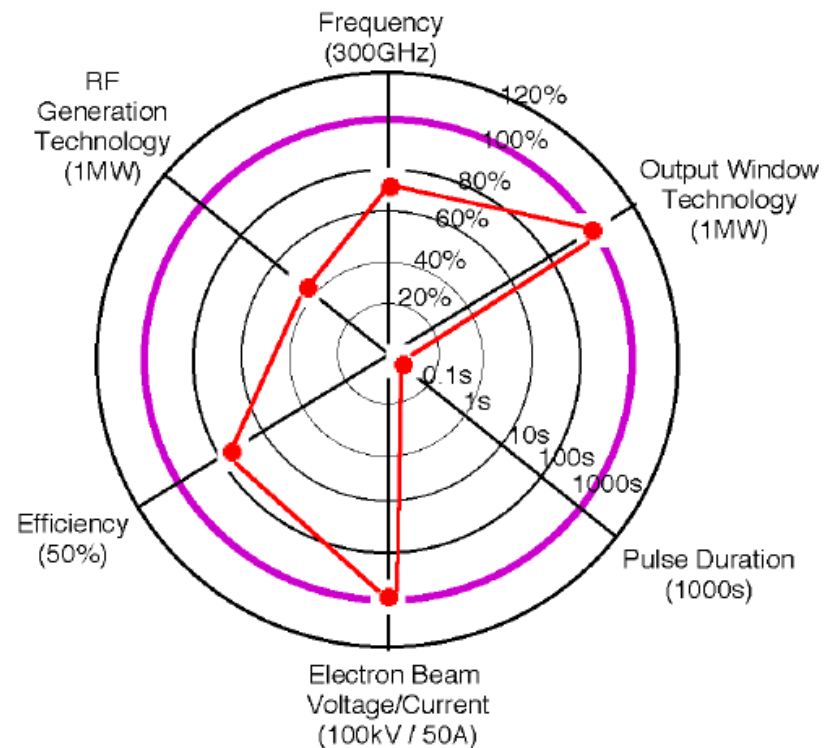
# Heating and CD system technology

- RF heating and CD technology in DEMO
  - Frequency of 300 GHz heat load: 5-20 MW/m<sup>2</sup>
  - Resonator enabling higher frequency oscillation, diamond window, frequency variable oscillator, etc.

**For ITER**



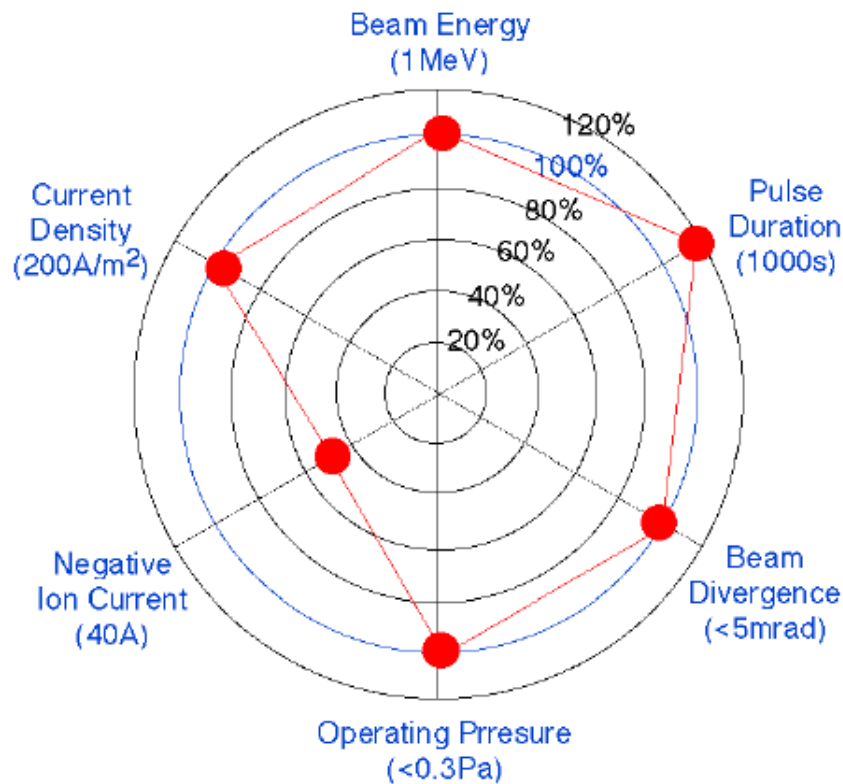
**For Demo Reactor**



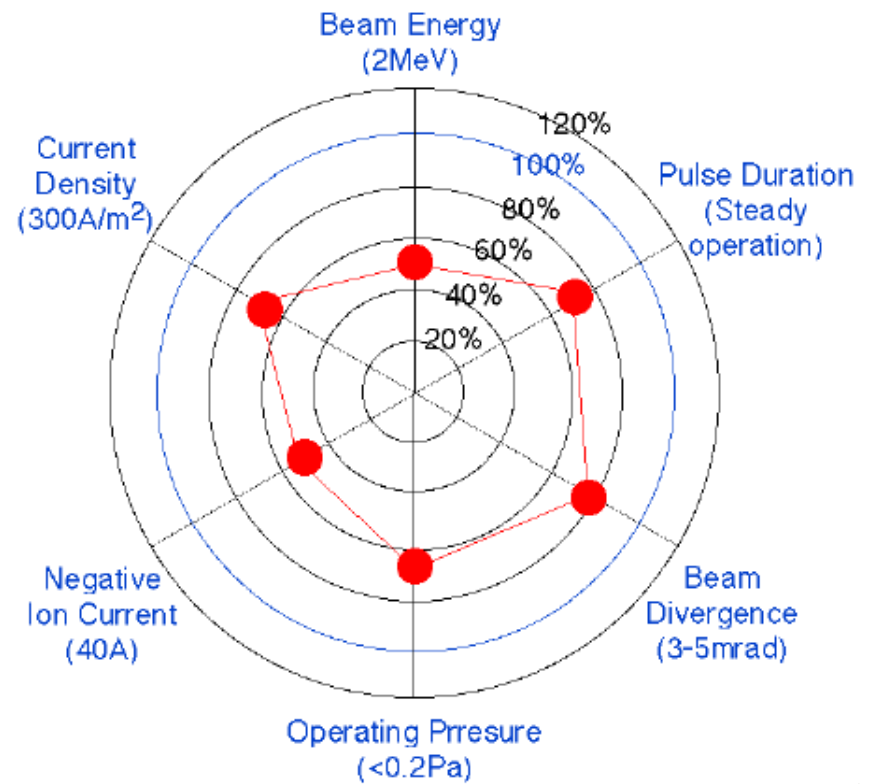
# Heating and CD system technology

- NBI technology in DEMO
  - Beam energy of 2 MeV
  - Maintenance-free negative ion source, plasma neutraliser with higher efficiency

**For ITER**



**For Demo Reactor**



# Tritium processing and safety technology

- Reliable tritium processing for steady and continuous long-term operation
- Safety for the power generation plant
- Production and security of necessary amounts of tritium
- Efficient tritium removal/recovery from contaminated wastes

## For ITER

## For Demo Reactor

container for  
250 g of  
tritium

Fuel Processing Cycle Technologies  
(Impurity Processing, Isotope  
Separation, Storage, etc.)

Fuel Processing Cycle Technologies  
(including Tritium Production for Initial  
Loading, Blanket Tritium Recovery)

System Integration  
Technologies  
(System Control  
and Reliability)

System Integration  
Technologies  
(System Control and  
Reliability)

Amount of  
Handling and  
Processing  
Tritium

Amount of Handling  
and Processing  
Tritium

Accounting  
and Analyzing  
Technologies

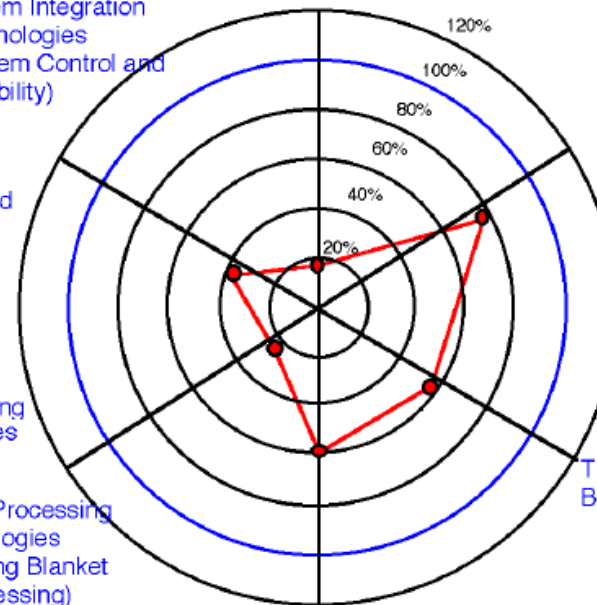
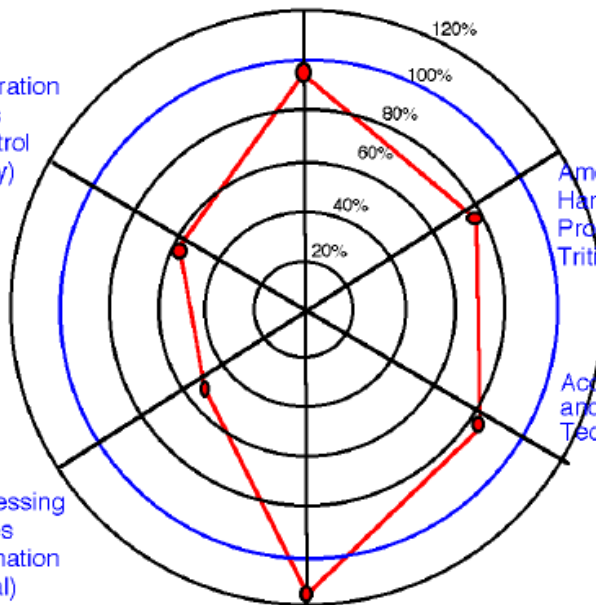
Tritium Inventory  
Balance Control

Waste Processing  
Technologies  
(Decontamination  
and Disposal)

Waste Processing  
Technologies  
(including Blanket  
Reprocessing)

Confinement and  
Removal Technologies

Confinement and Removal Technologies  
(including safety Technologies for Blanket  
and Power Generation System)



# Tritium processing and safety technology

## • Tritium Export

- **Total tritium to be received:**
  - ~ 29 kg due to tritium decay
  - Decay rate: 5.47 %/y (Half life; 12.3 y)
- ITER Tritium Plant will be ready by 2016
- Tritium available worldwide: ~ 20 kg (2006, Canada OPG) (+ Korea WTRF)
  - ITER Tritium credit: \$30M/kg
  - Market value: \$100M-\$200M/kg
- Only one supplier for ITER written on the ITER documents now: Canada
- Canada OPG sells ~ 0.1 kg/yr for other purposes.
- **There is no other kg's order civilian tritium source at all.**
- WTRF can produce more than 0.7 kg/yr from this year (2006).
- We have Tritium and good reason to supply.
  - Korea is a partner for ITER, Canada is not.
  - Korea is to procure the Tritium Storage and Delivery System for ITER.

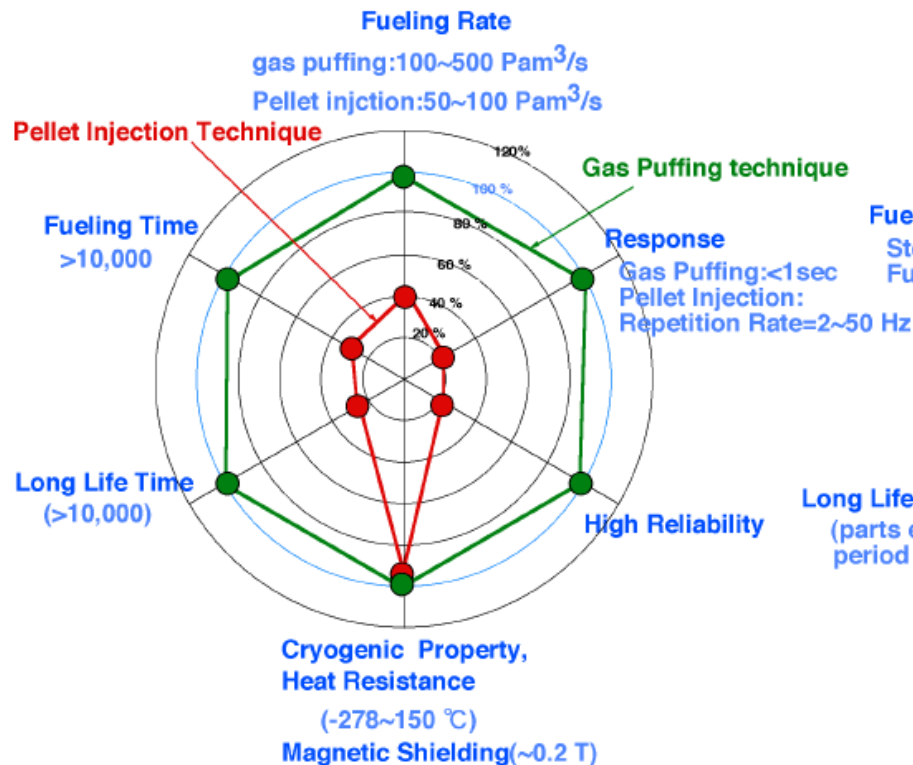


*C. S. Kim, "Tritium Export Preparation for ITER Operation and Fusion Applications",  
May 25, 2006, NFRI*

# Fuelling and vacuum pumping technology

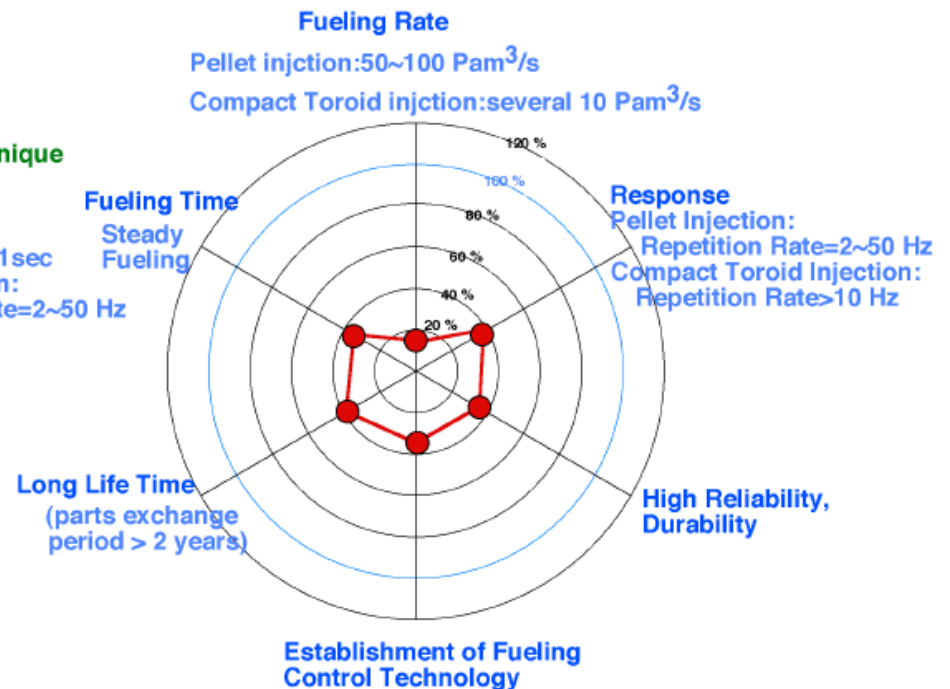
## For ITER

- \* development of gas puffing technique
- \* development of pellet injection technique



## For Demo Reactor

- \* development of center fuelling technique



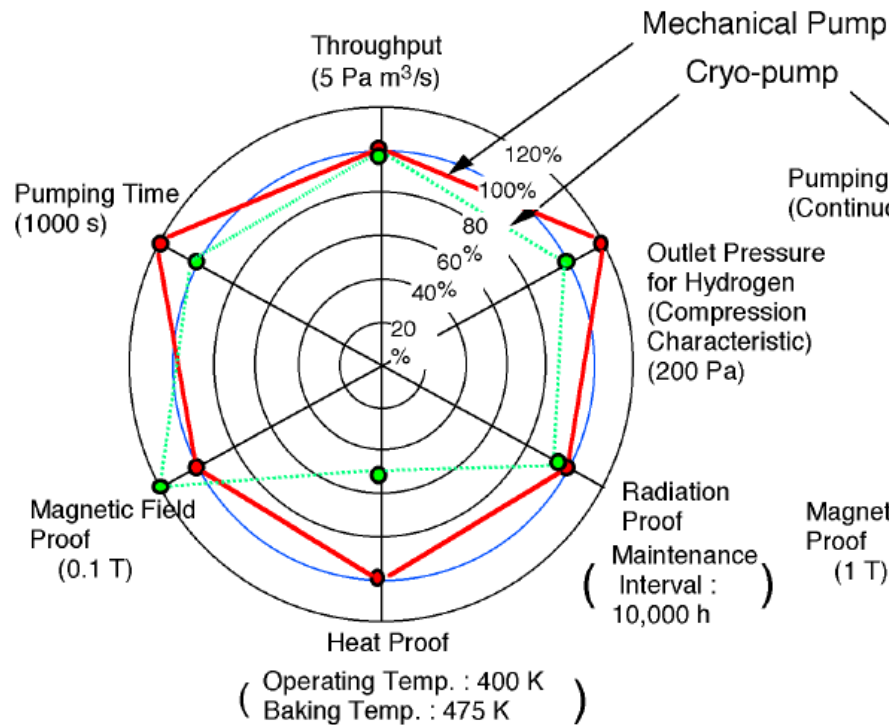
**HW: compact toroid injection?**



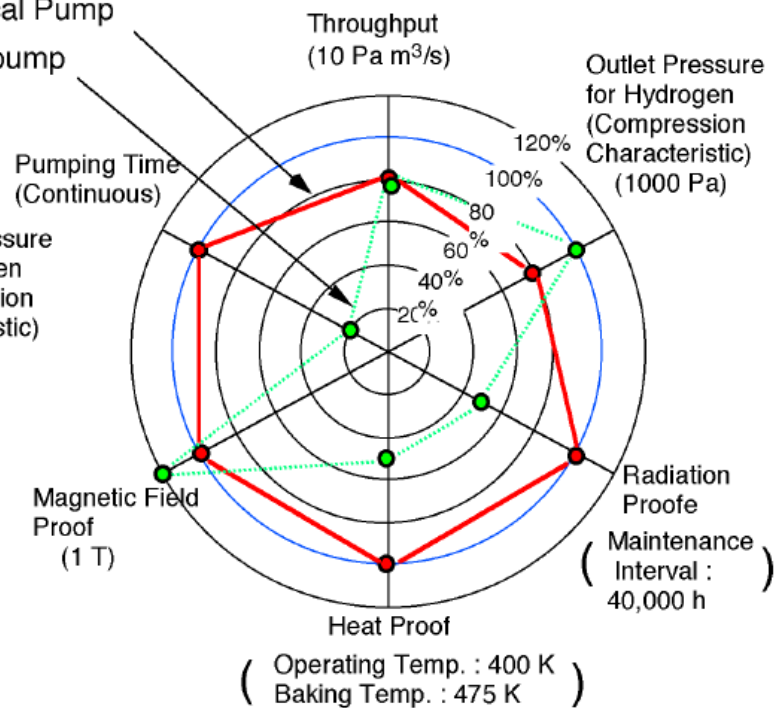
# Fuelling and vacuum pumping technology

- Mechanical pump: metallic rotors requiring magnetic shield, low tritium inventory
- Cryopump: not disturbed by a magnetic field, operation has to be stopped periodically for regeneration of the cryopanel.

For ITER



For Demo Reactor



vacuum leak detection method

Assuming a ceramic pump driven by a compressed gas in a Demo reactor.

# Diagnostics technology

## For ITER

### \* Development of Diagnostic Elements

Ceramics Insulation,  
Optical Elements (Mirror/Reflector, Window Materials,  
Optical Fiber),  
Sensors (Magnetic Probe, Bolometer,  
Pressure Gauge, etc.),  
Electric Cable

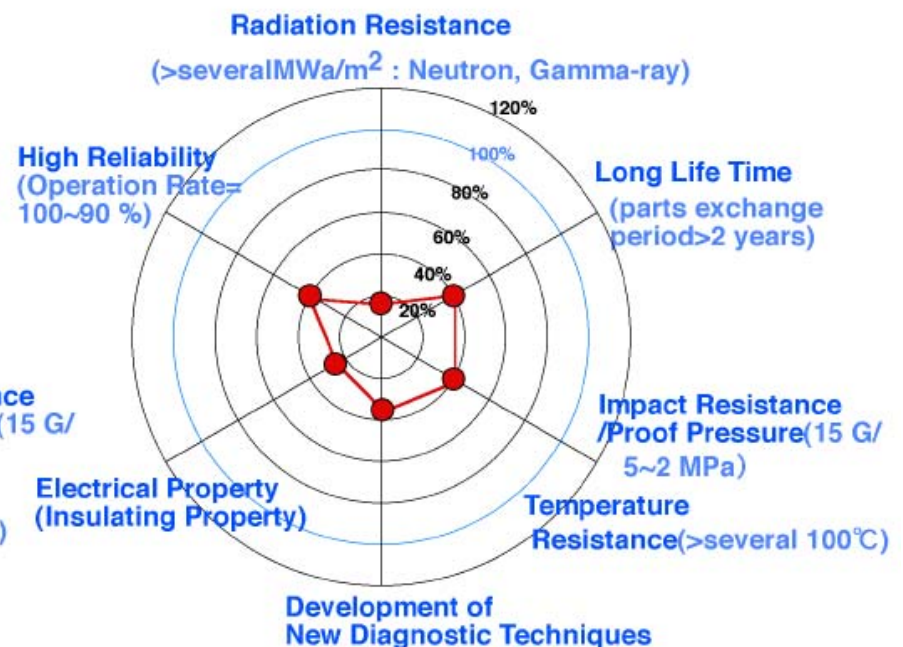
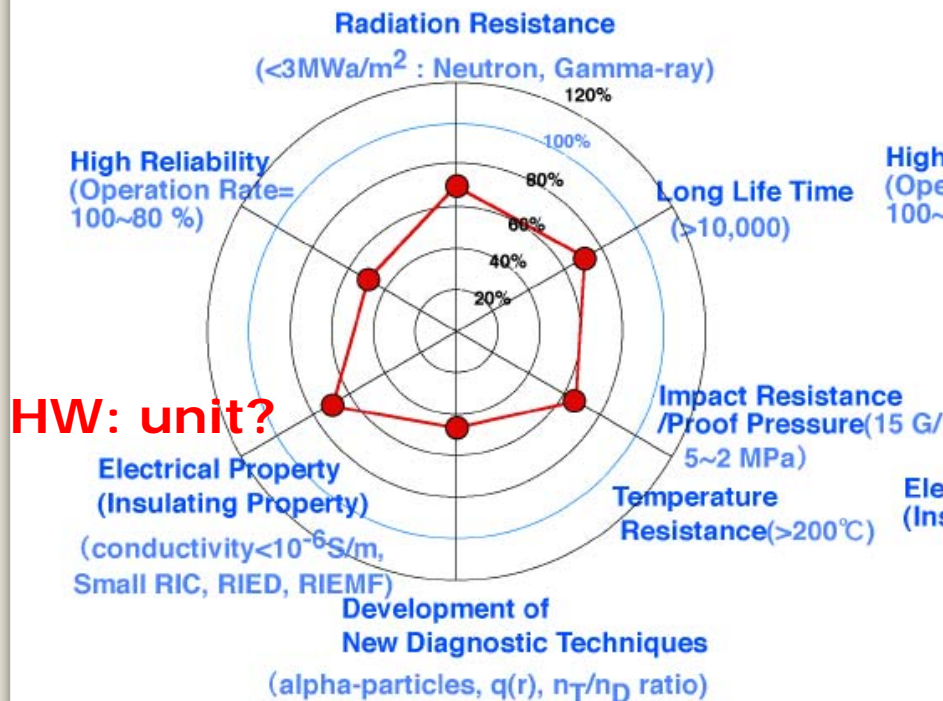
### \* Development of Prototype

Sensors, Vacuum Seals for Diagnostic Window,  
Optical fiber/ Electric Cable Feedthroughs, etc.

## For Demo Reactor

### \* Development of Diagnostic Elements/Prototype

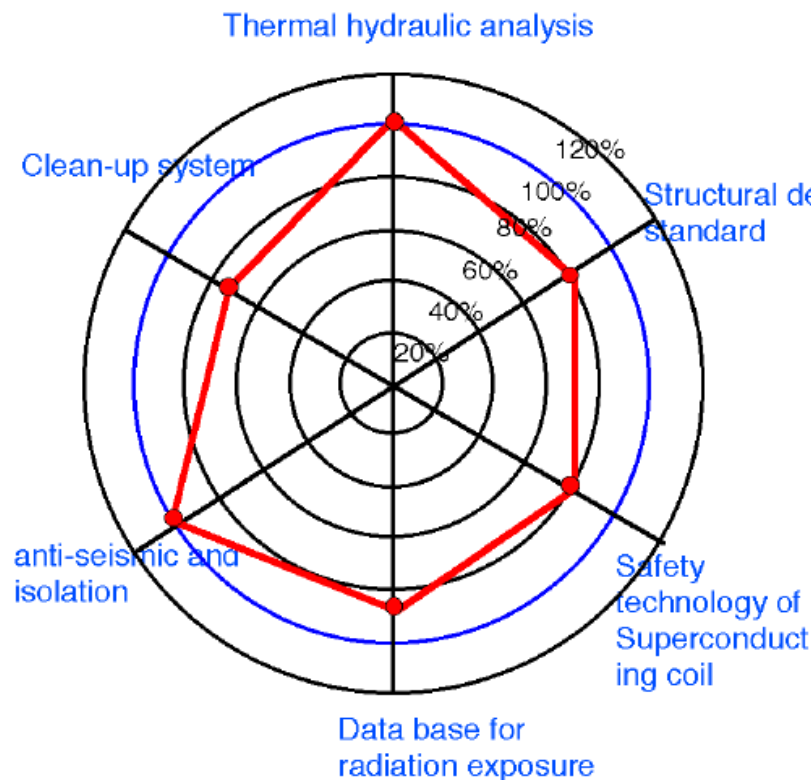
Development of Advanced Materials,  
Heavy Irradiation Tests with  $<20$  dpa



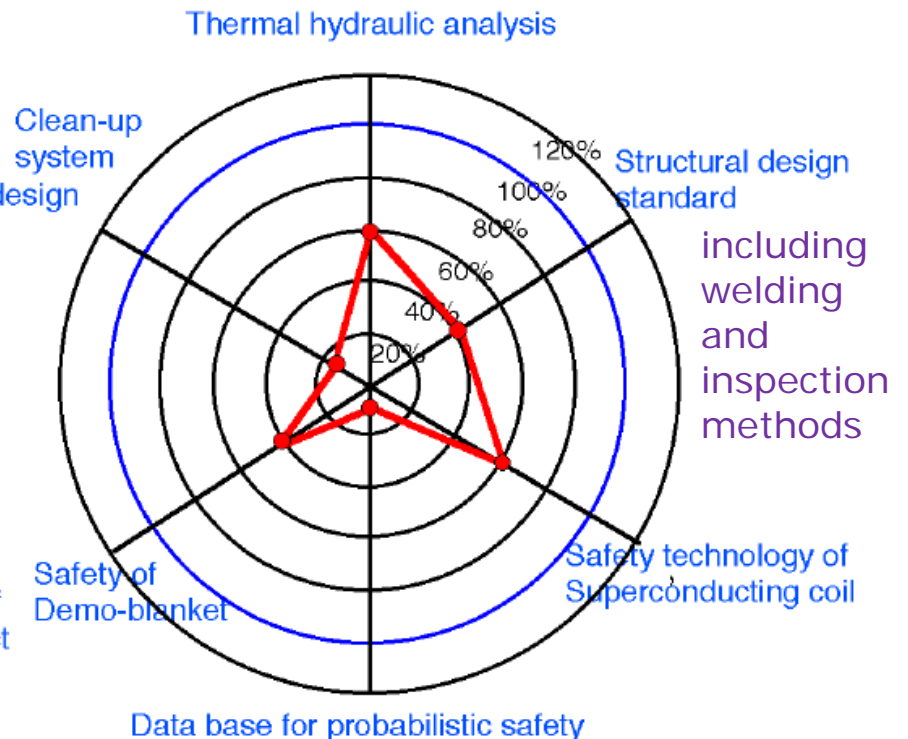
# Safety technology

- Improvement of the safety system reliability for abnormal events of the cooling system due to high coolant temperature, high heat flux, and high neutron flux for power generation, and the improvement of social receptivity of the programme by reationalisation and passiveness.

For ITER



For Demo Reactor



# Materials development

For ITER  
(Austenitic Steel)

For Demo Reactor  
(Reduced Activation Ferritic  
/ Martensitic Steels)

