

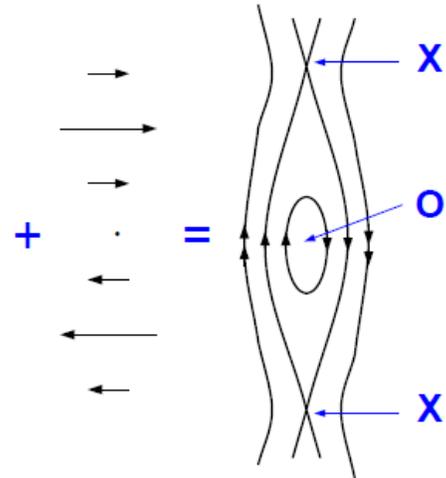
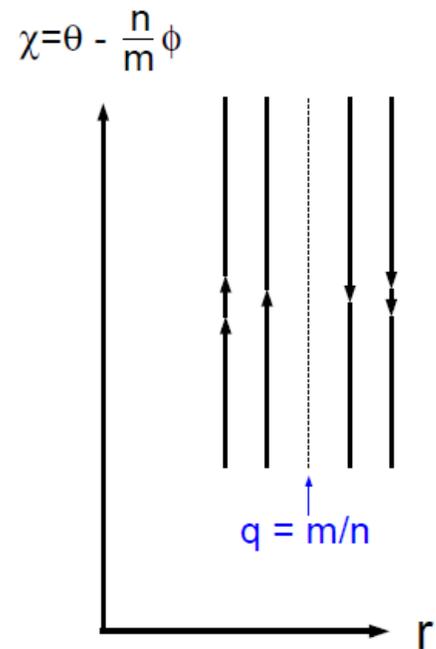
# Introduction to Nuclear Fusion

Prof. Dr. Yong-Su Na

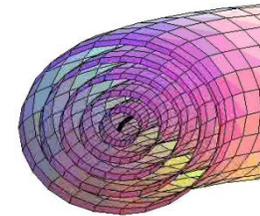
# Resistive MHD instabilities in a Tokamak

# Resistive MHD Instabilities

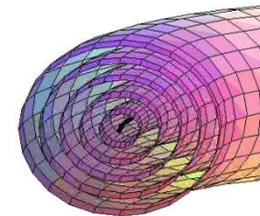
- growing more slowly compared with the ideal instabilities ( $10^{-4}$ - $10^{-2}$  s)
- resulting from the diffusion or tearing of the magnetic field lines relative to the plasma fluid
- destroying the nested topology of the magnetic flux surfaces



$$\Delta' = \frac{1}{\psi} \left[ \frac{d\psi}{dr} \Big|_{r=r_s^+} - \frac{d\psi}{dr} \Big|_{r=r_s^-} \right] > 0$$



Flux conservation  
Topology unchanged

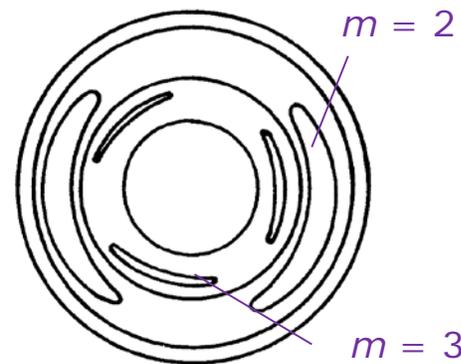


Reconnection of field lines  
Topology changed

# Resistive MHD Instabilities

## • Tearing Modes

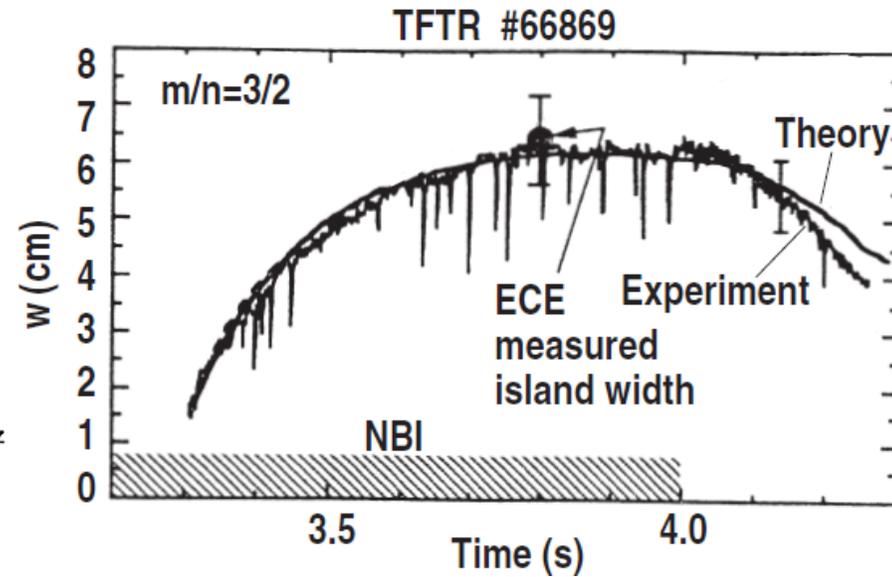
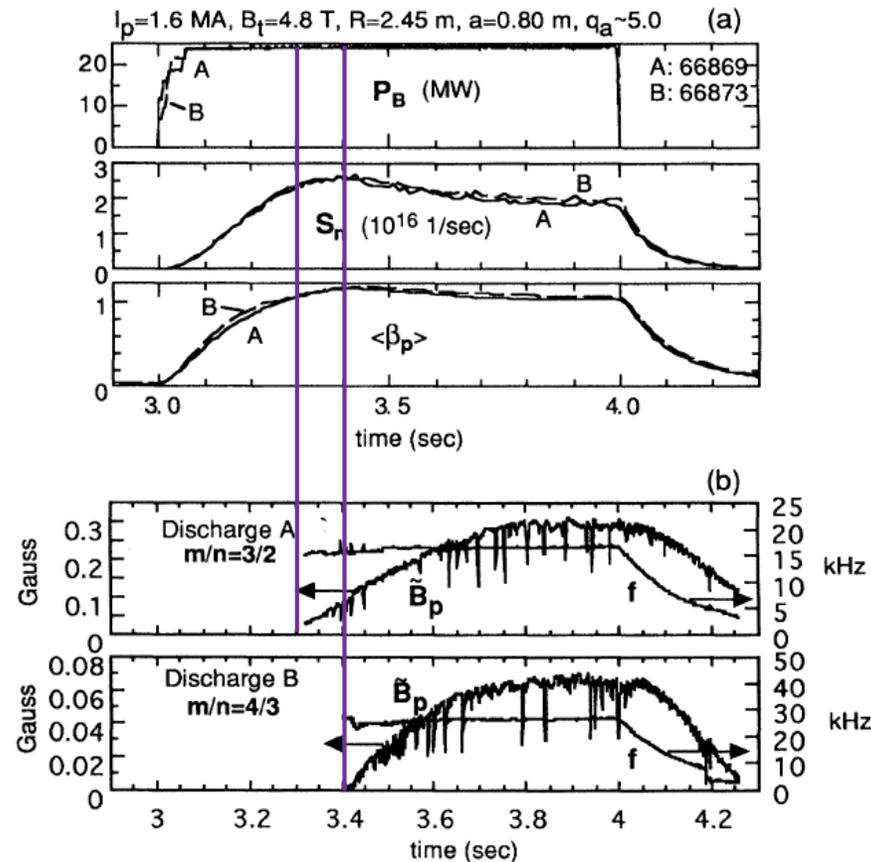
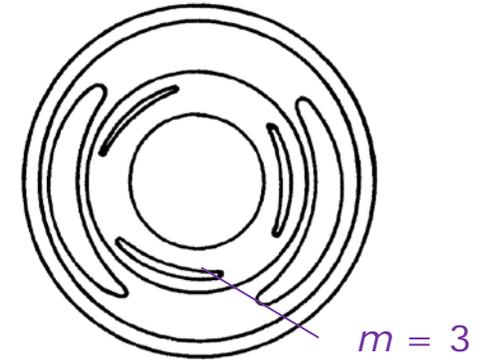
- resistive internal kink modes ( $m \geq 2$ )
- driven by perturbed  $\mathbf{B}$  induced by current layer ( $\nabla J$ ) in plasmas
- magnetic island formation
- more tolerable and lower than ideal modes
- unstable region reduced as sharpness of the current profile, closeness of the wall to the plasma, shear increases
- stability condition:  $q_0 > 3$



# Resistive MHD Instabilities

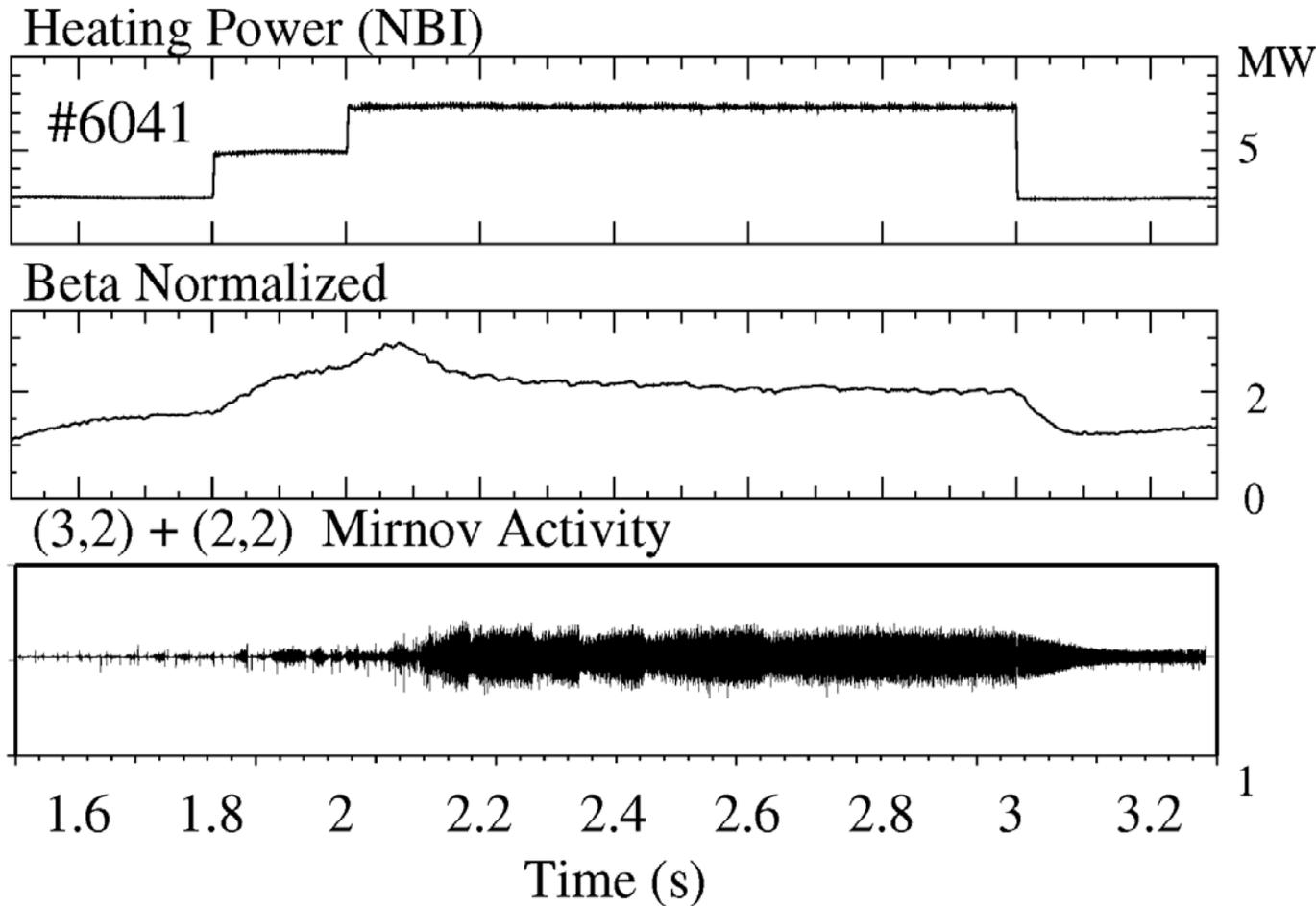
- Neoclassical Tearing Modes (NTMs)

- $\Delta' < 0$
- Predicted theoretically first, observed experimentally in 1995



# Resistive MHD Instabilities

- Neoclassical Tearing Modes (NTMs)



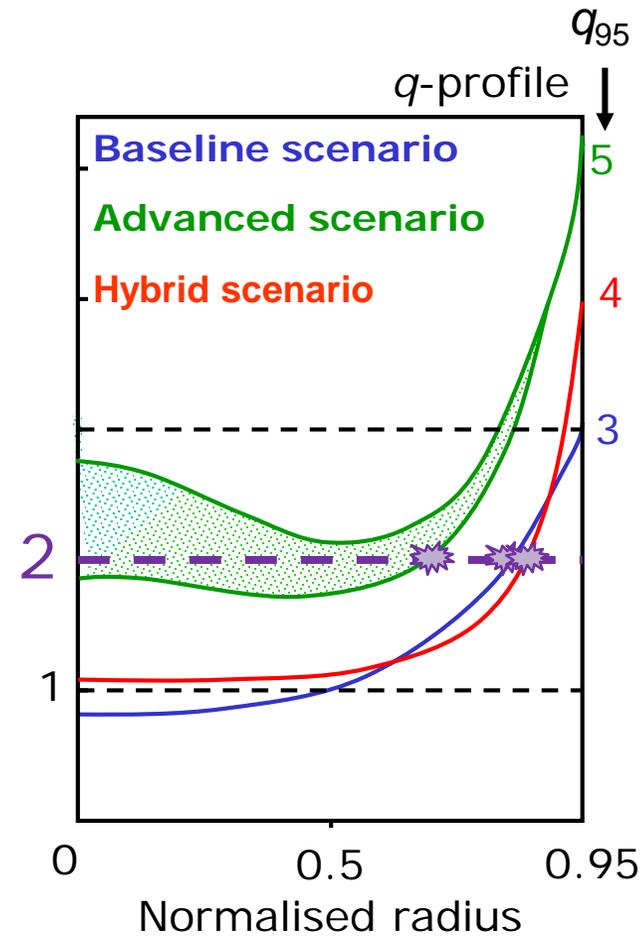
$$\beta_t = \frac{p}{B_t^2 / 2\mu_0} = \beta_N \frac{I_p}{aB_t}$$

# Resistive MHD Instabilities

- Neoclassical Tearing Modes (NTMs)

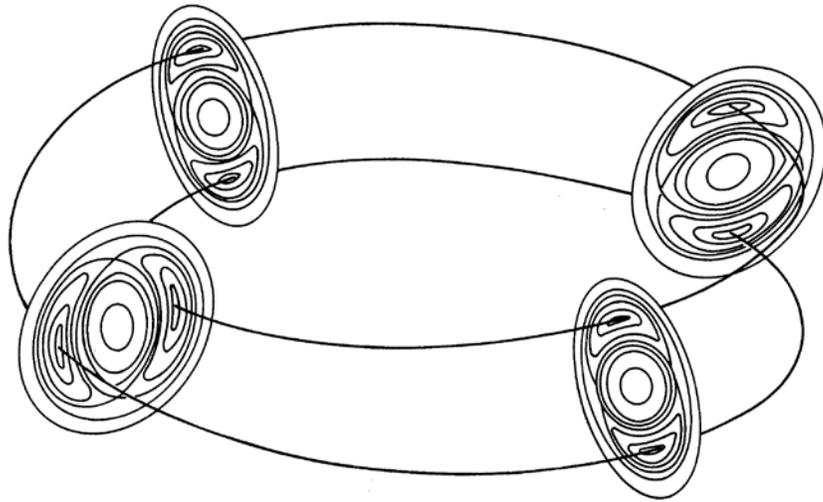


$q = m/n = 2$  surface ( $m = 2, n = 1$ )

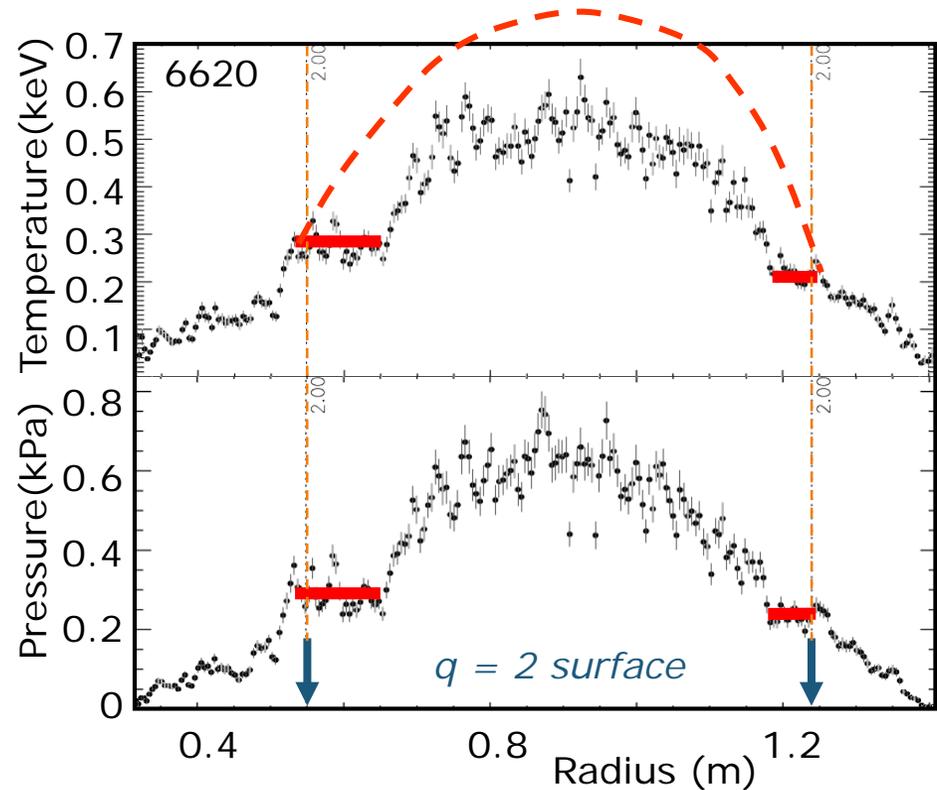


# Resistive MHD Instabilities

- Neoclassical Tearing Modes (NTMs)



$q = m/n = 2$  surface ( $m = 2, n = 1$ )



- Pressure flattening across magnetic islands due to large transport coefficients along magnetic field lines

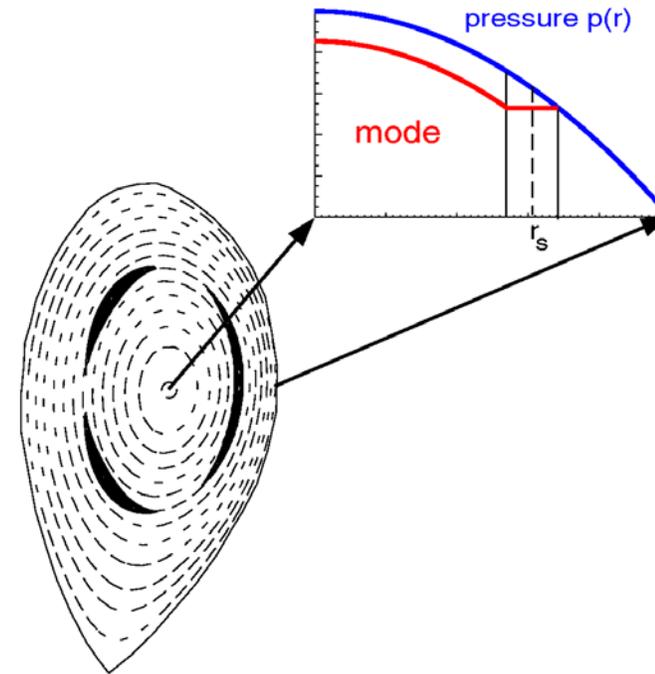
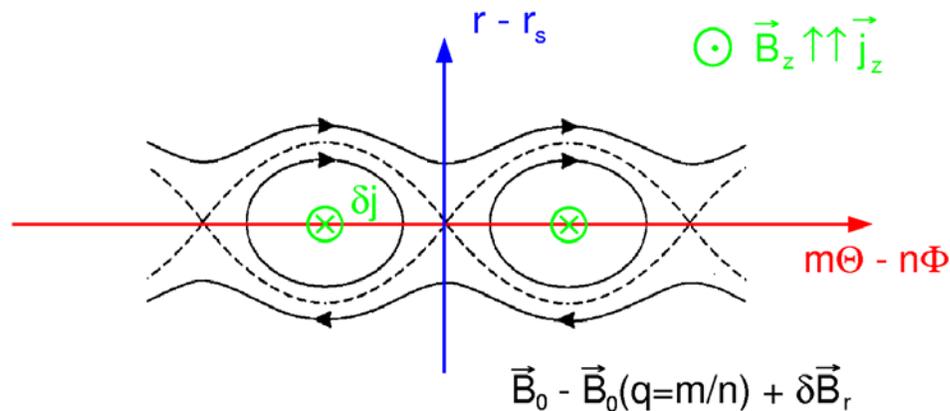
# Resistive MHD Instabilities

- Neoclassical Tearing Modes (NTMs)

- pressure gradient drives plasma current (Bootstrap current):

$$j_{BS} \propto \nabla p$$

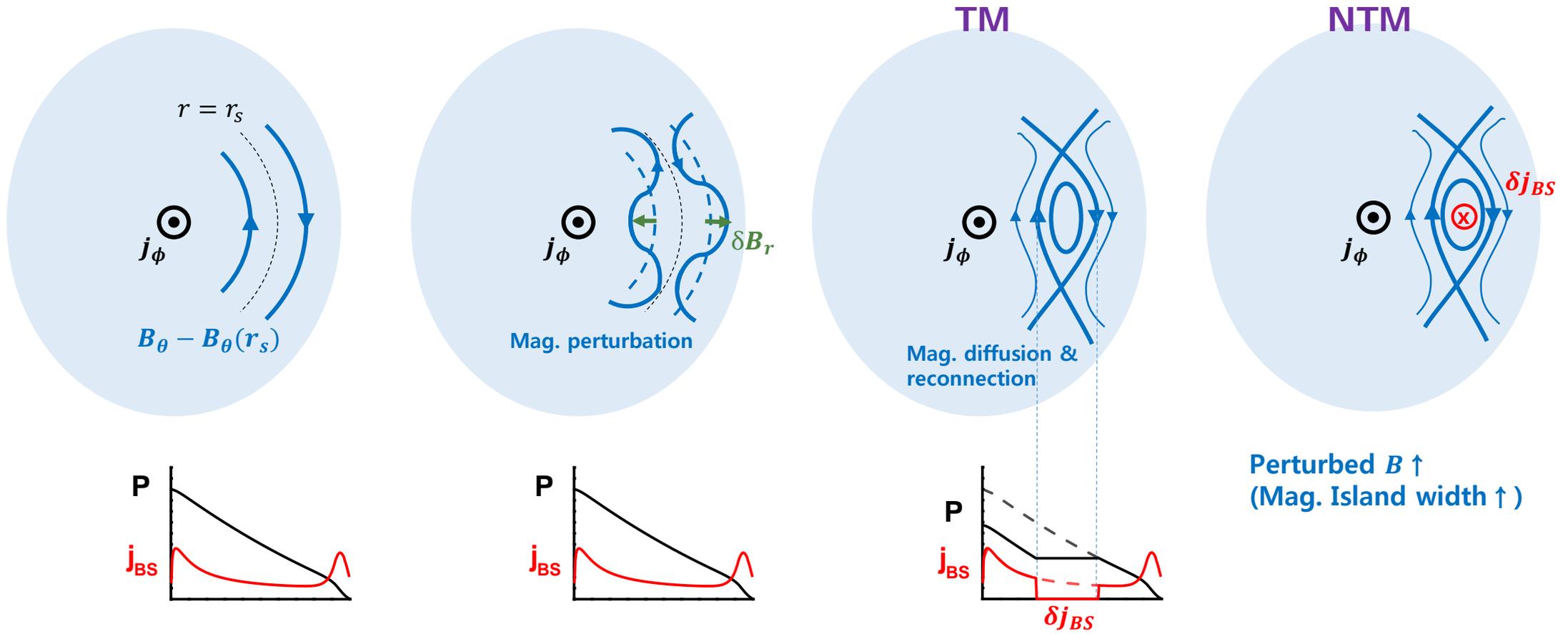
- inside islands  $\nabla p$  flattened  $\rightarrow j_{BS}$  vanished



- Loss of BS current inside magnetic islands acts as helical perturbation current driving the islands – so once seeded, island is sustained by lack of bootstrap current.

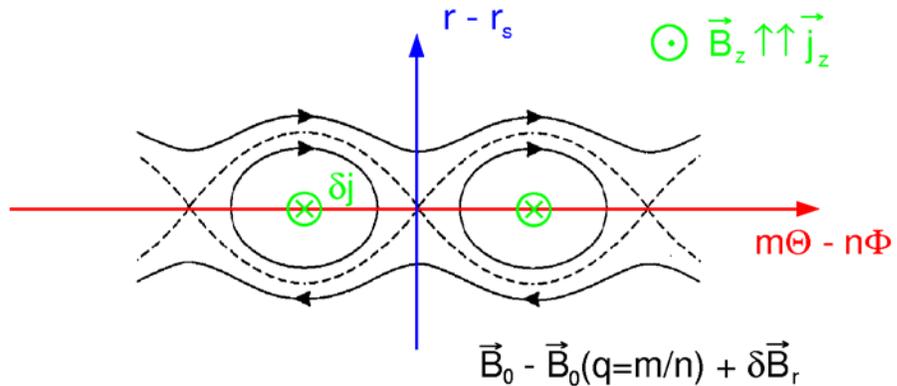
# Resistive MHD Instabilities

- Neoclassical Tearing Modes (NTMs)

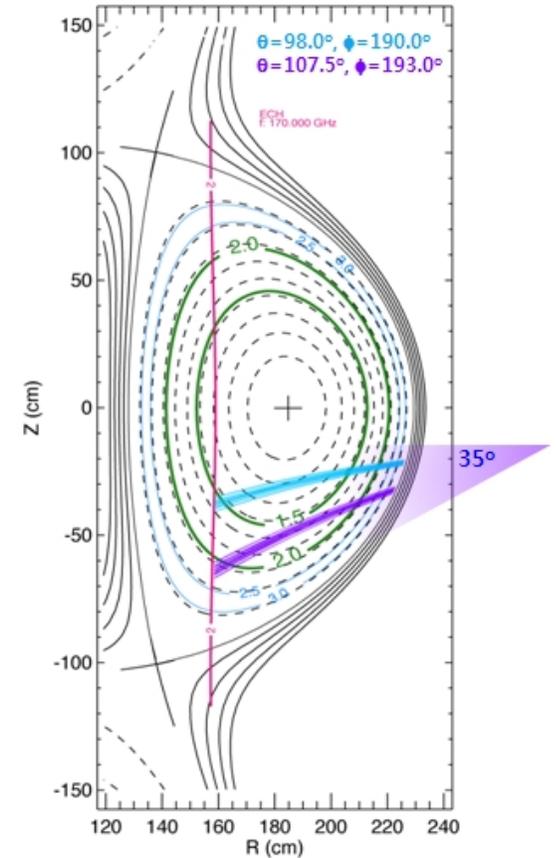
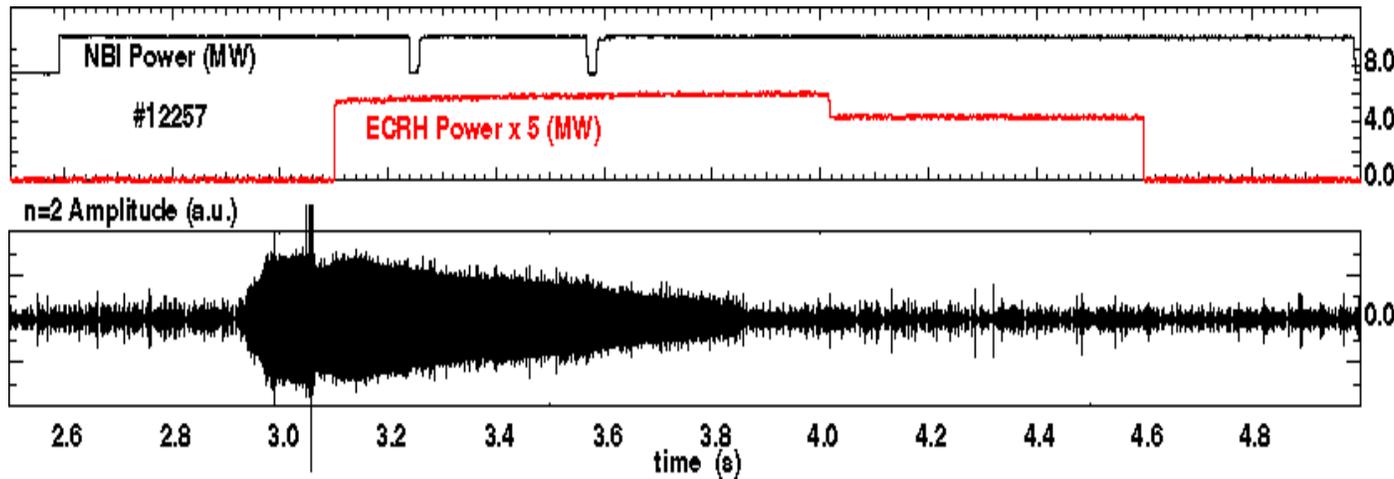


# Resistive MHD Instabilities

- Neoclassical Tearing Modes (NTMs)



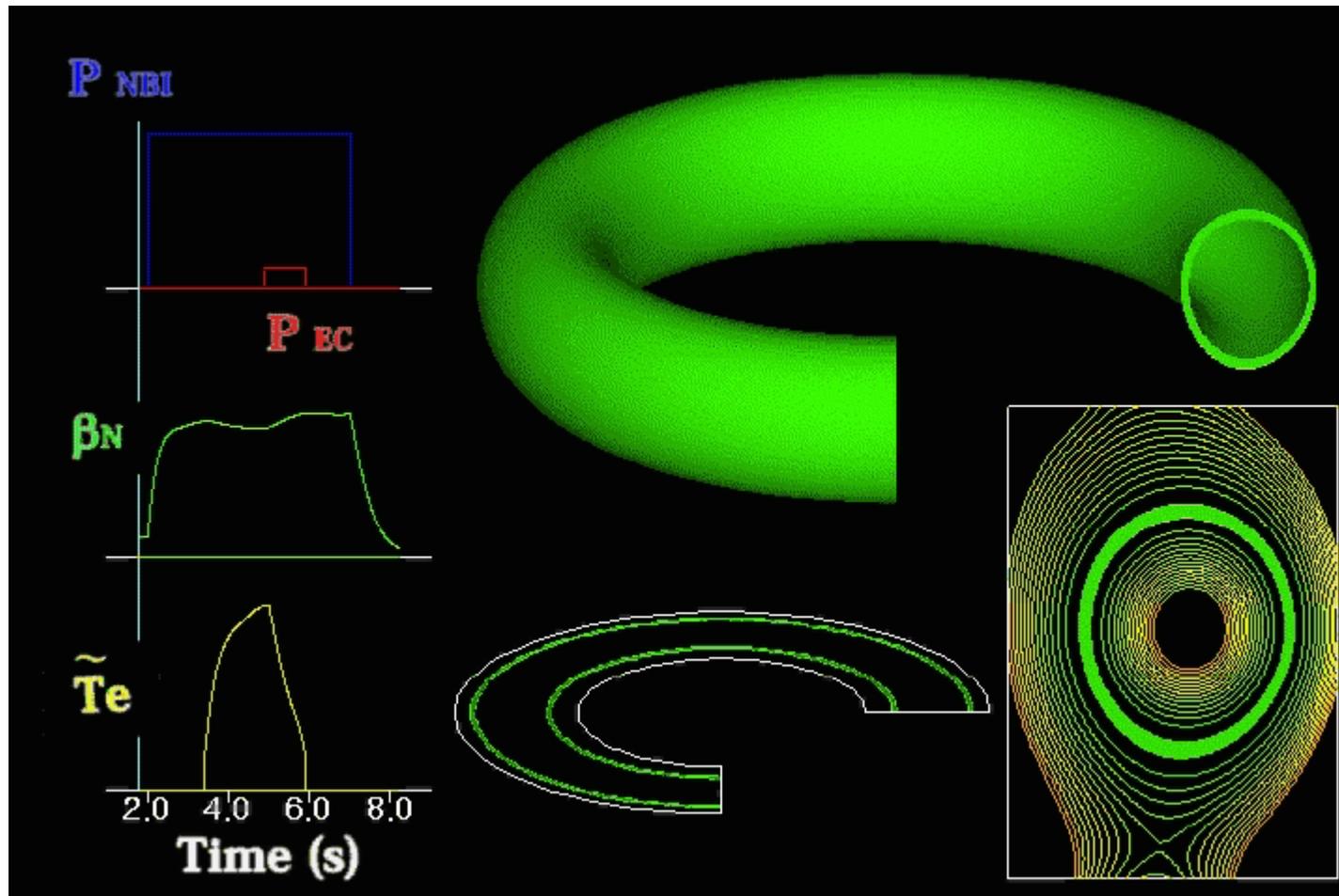
- Missing bootstrap current inside island can be replaced by localised external current drive.



- complete stabilisation in quantitative agreement with theory.

# Resistive MHD Instabilities

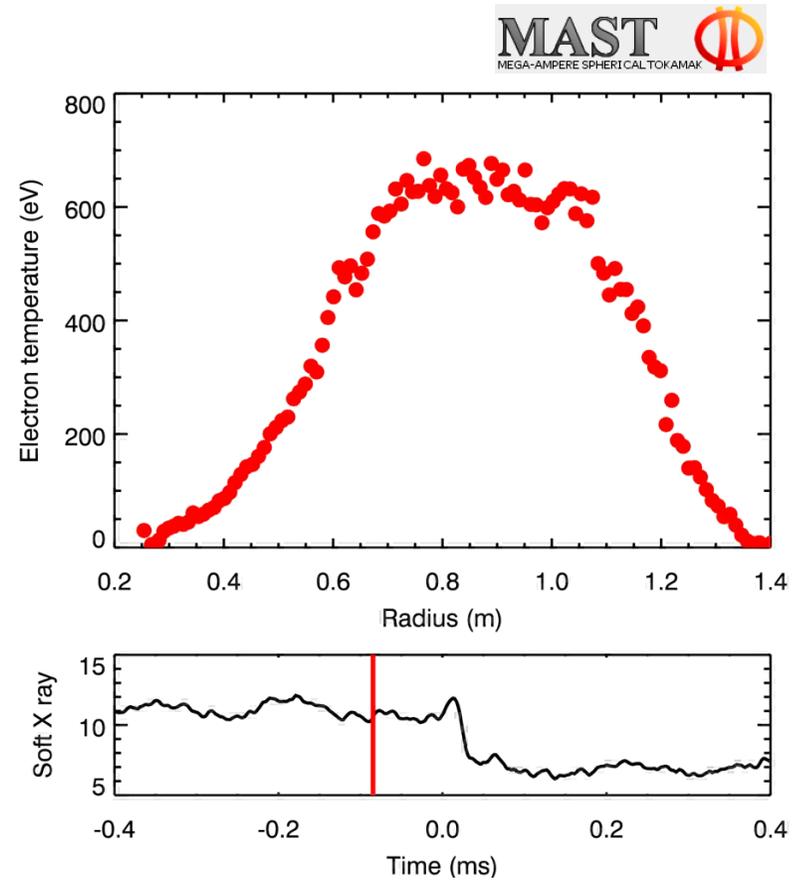
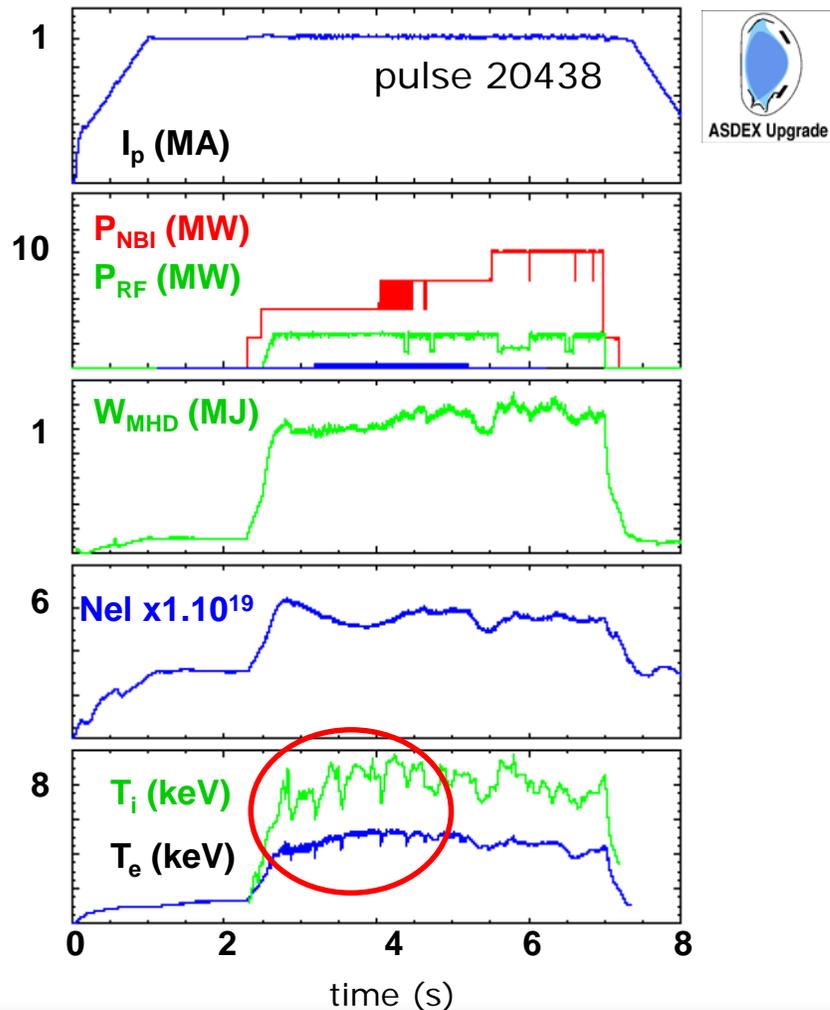
- Neoclassical Tearing Modes (NTMs)



# Complex non-linear instabilities in a Tokamak

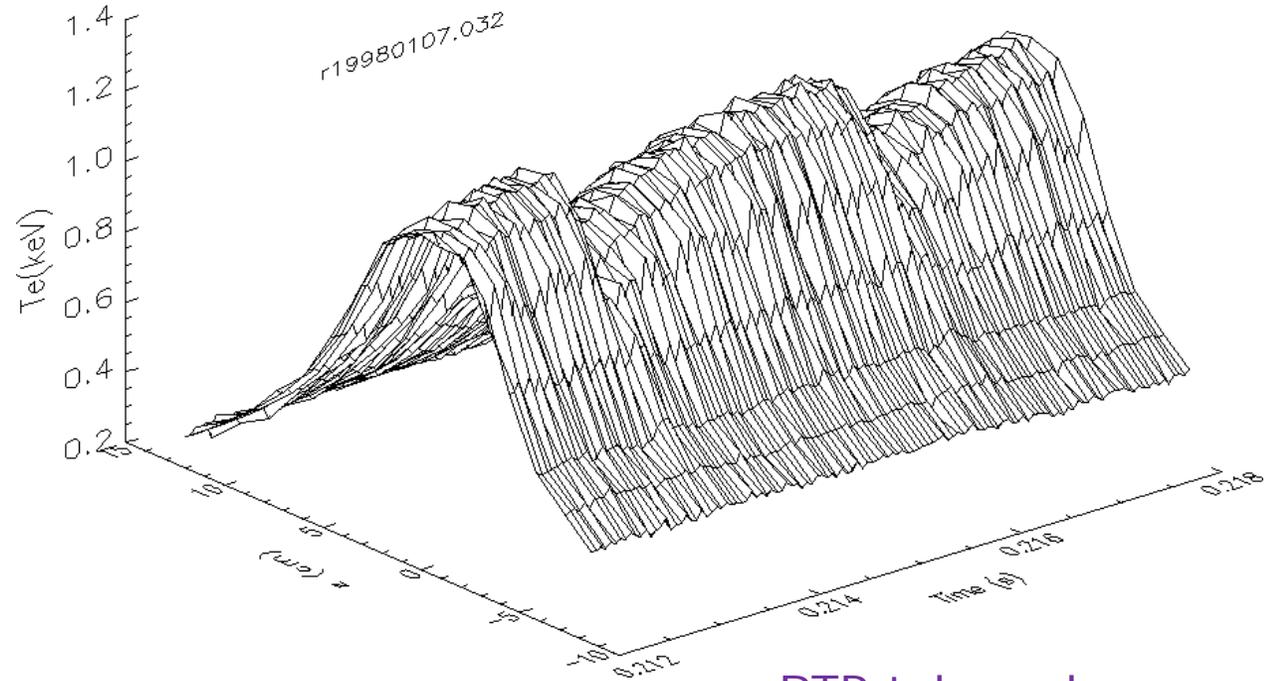
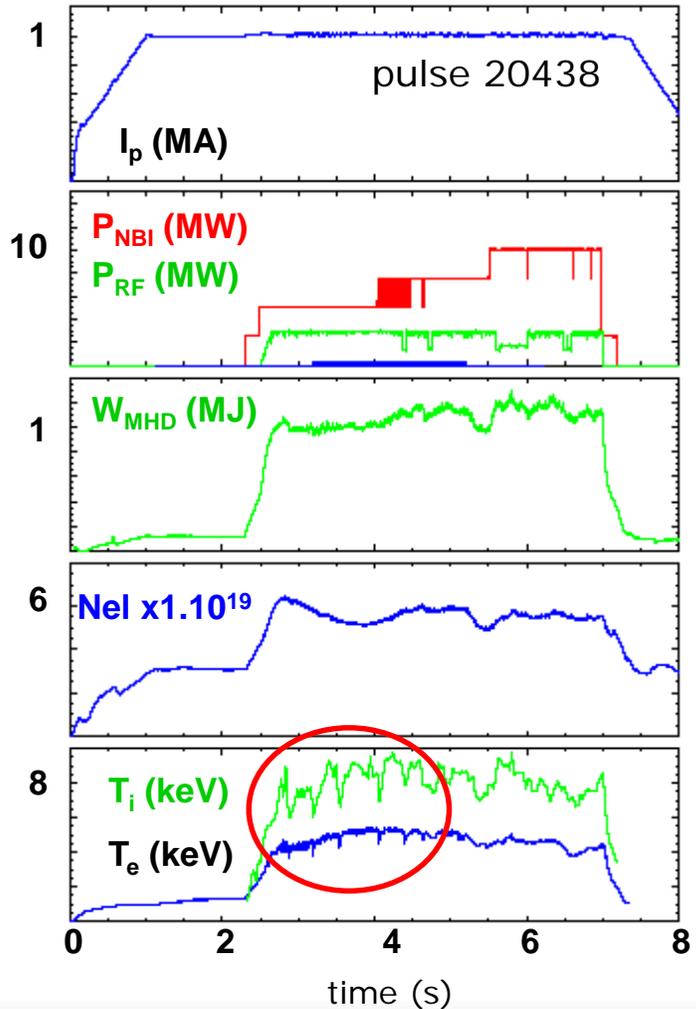
# Non-linear MHD Instabilities

- Nonlinear low- $n$  internal modes: Sawtooth



# Non-linear MHD Instabilities

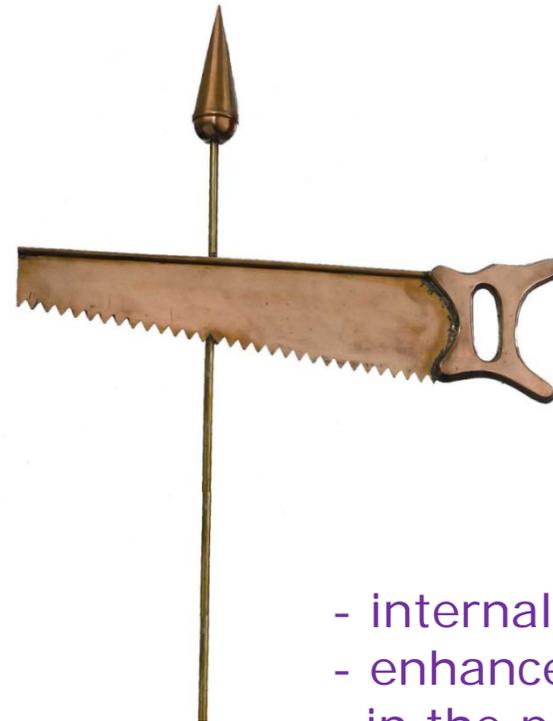
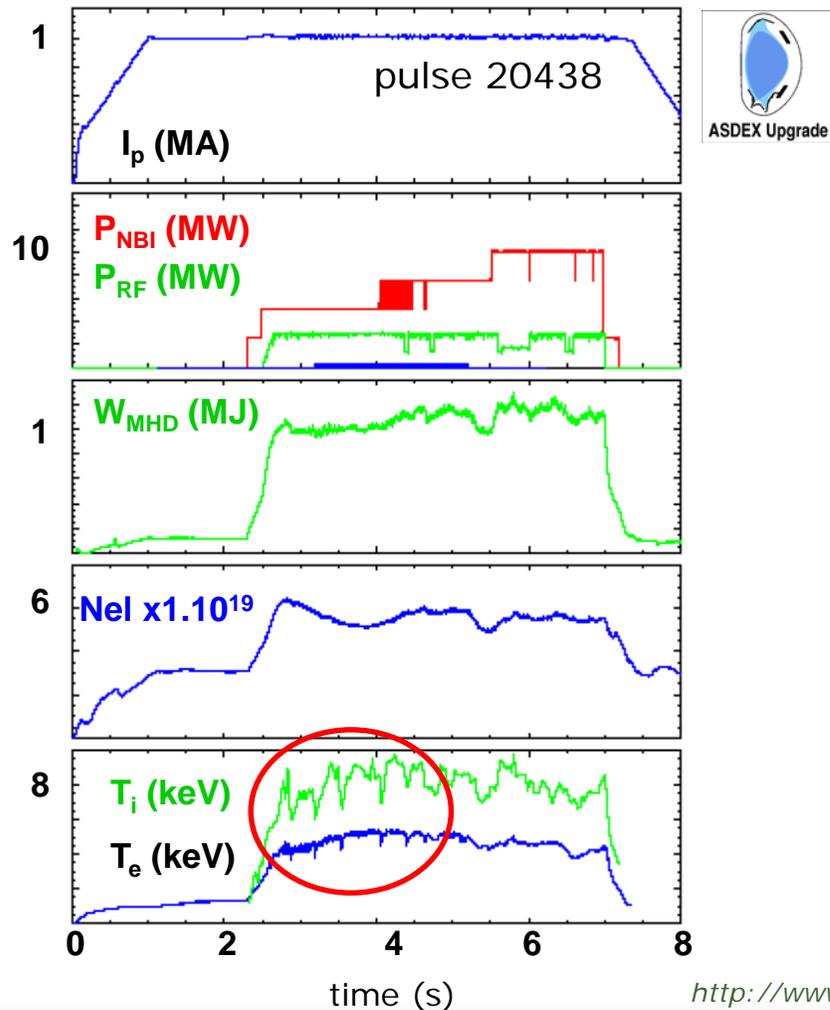
- Nonlinear low- $n$  internal modes: Sawtooth



RTP tokamak

# Non-linear MHD Instabilities

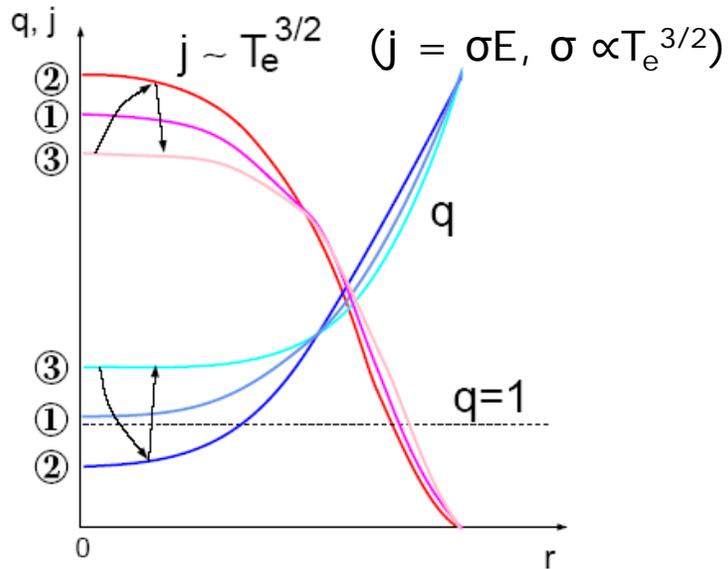
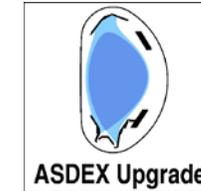
- Nonlinear low- $n$  internal modes: Sawtooth



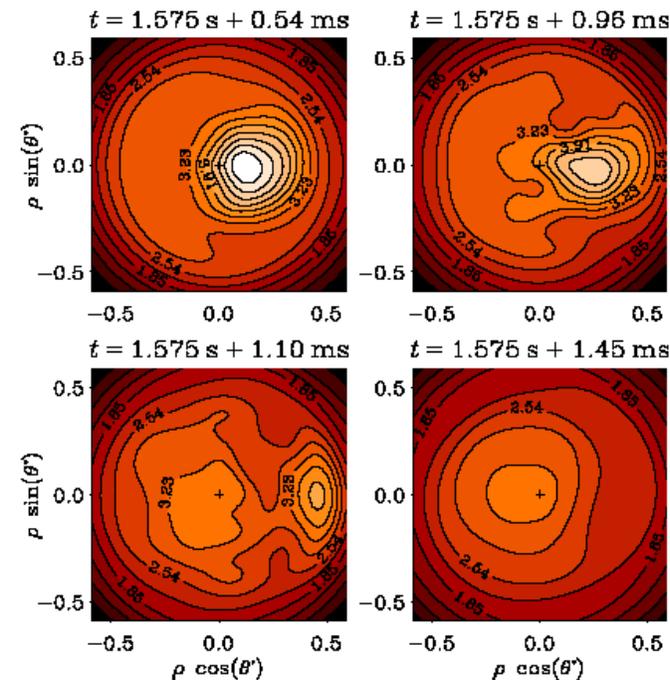
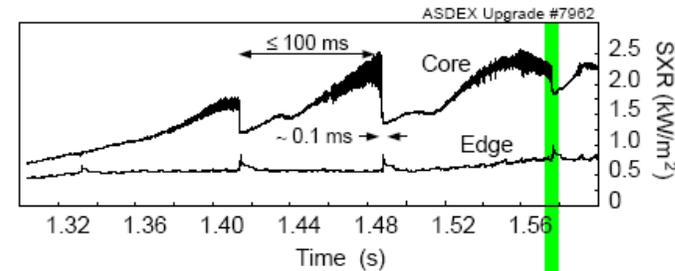
- internal (minor) disruption
- enhanced energy transport in the plasma centre

# Non-linear MHD Instabilities

## • Nonlinear low- $n$ internal modes: Sawtooth

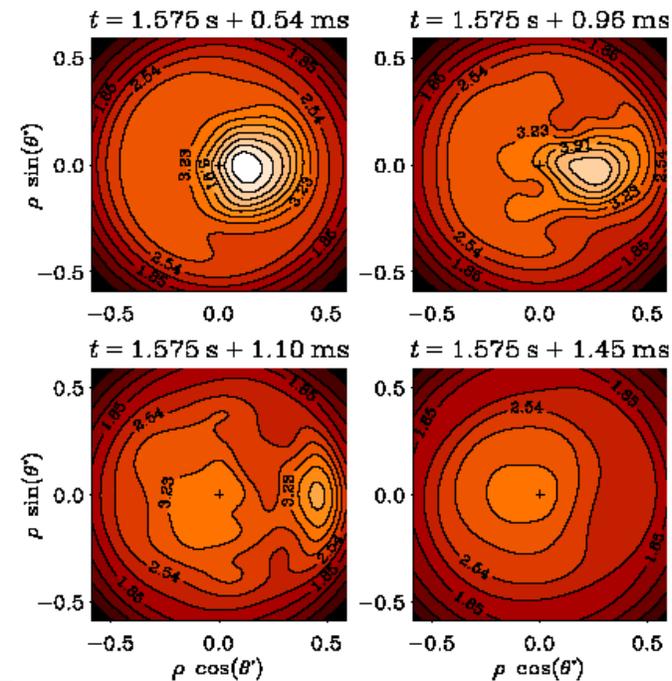
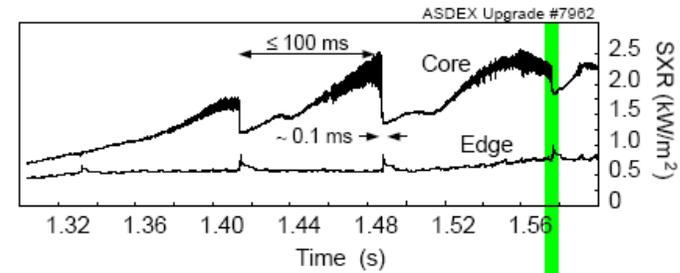
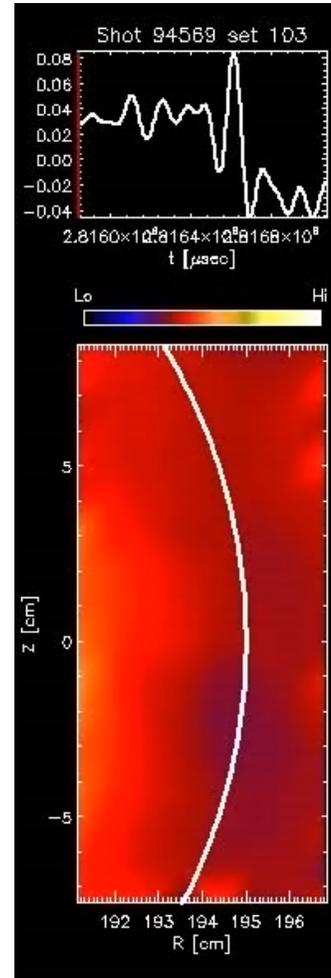
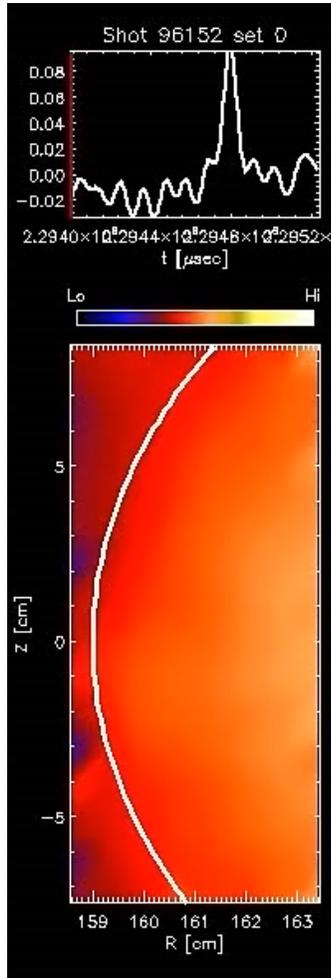
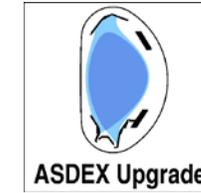


1.  $T(0)$  and  $j(0)$  rise
2.  $q(0)$  falls below 1  
→ kink instability grows
3. Fast reconnection event:  
 $T, n$  flattened inside  $q = 1$  surface  
 $q(0)$  rises slightly above 1  
kink stable



# Non-linear MHD Instabilities

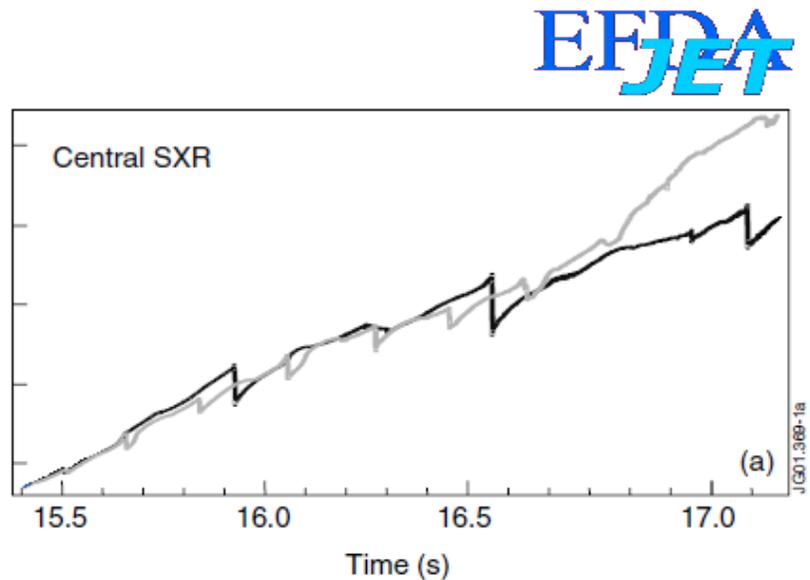
- Nonlinear low- $n$  internal modes: Sawtooth



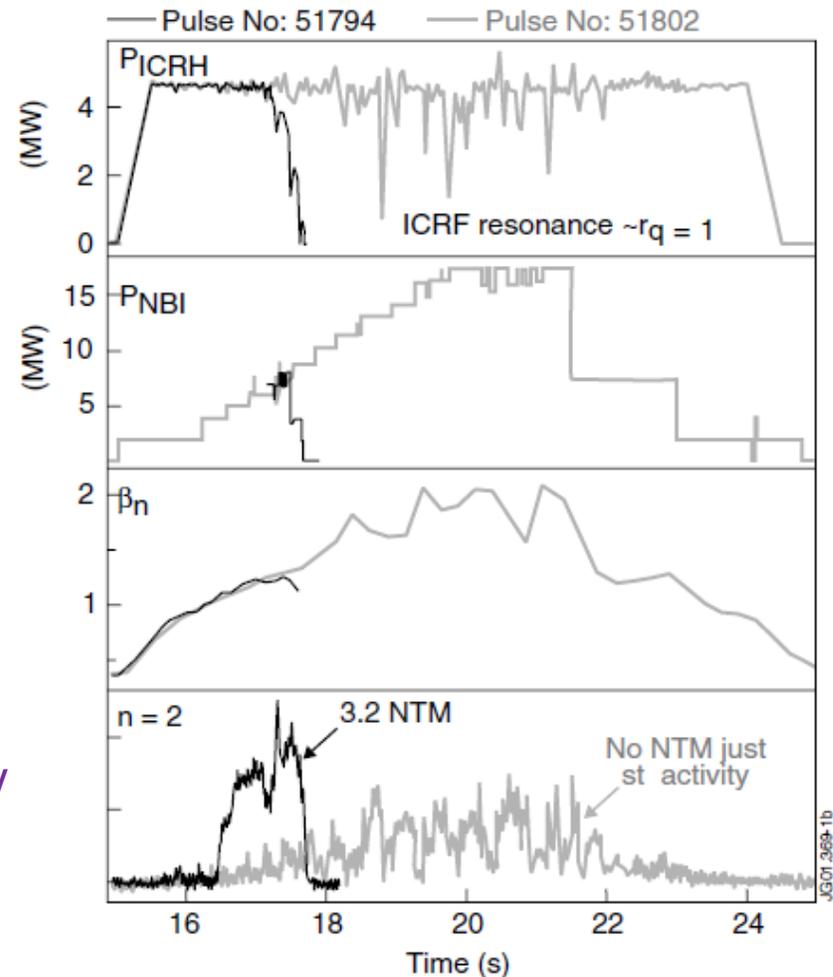
Courtesy of  
H.K. Park  
(UNIST)

# Non-linear MHD Instabilities

- Nonlinear low- $n$  internal modes: Sawtooth



- Increased sawtooth period due to stabilisation by fast ions produced by ICRH leads to the triggering of  $n = 2$  NTM activity which causes a termination of the discharge.



# Non-linear MHD Instabilities

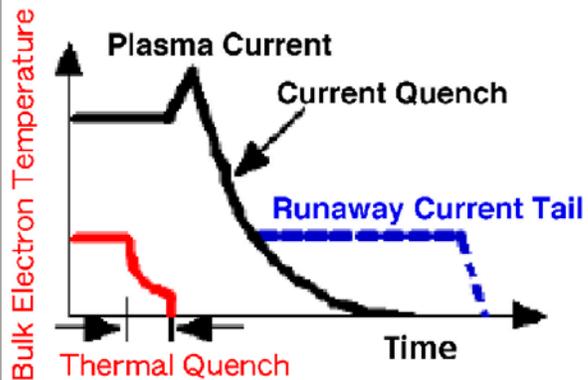
- Major Disruption



# Non-linear MHD Instabilities

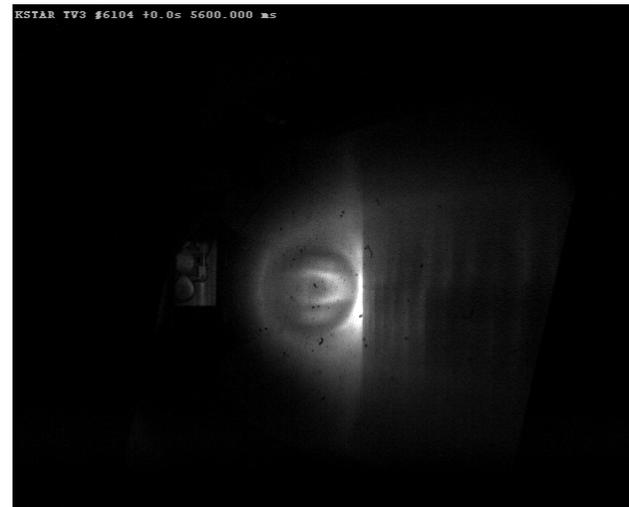
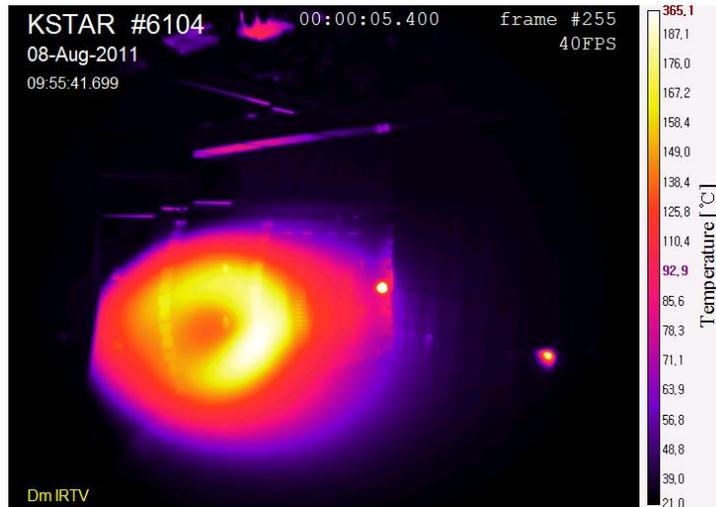
## • Major Disruption

- Disruptions are fast ( $\sim 1$  ms) global instabilities that may arise in magnetic confinement fusion devices that use plasma current for confinement such as tokamak.
- Termination of confinement, uncontrolled loss of thermal and magnetic energy
  - shift of the plasma column
  - heat load damage to plasma facing components (PFCs)
  - large mechanical stresses from  $\mathbf{J} \times \mathbf{B}$  forces during current quench
  - rapid cooling of the plasma  $\rightarrow$  increase of resistivity
  - increase of loop voltage  $\rightarrow$  runaway electrons (0.1-10 MeV) through avalanche amplification, resulting in a  $> 5$  MA of relativistic electron beam
    - $\rightarrow$  deep penetration of materials ( $\sim$ cm)



# Non-linear MHD Instabilities

- Major Disruption



- Synchrotron radiation

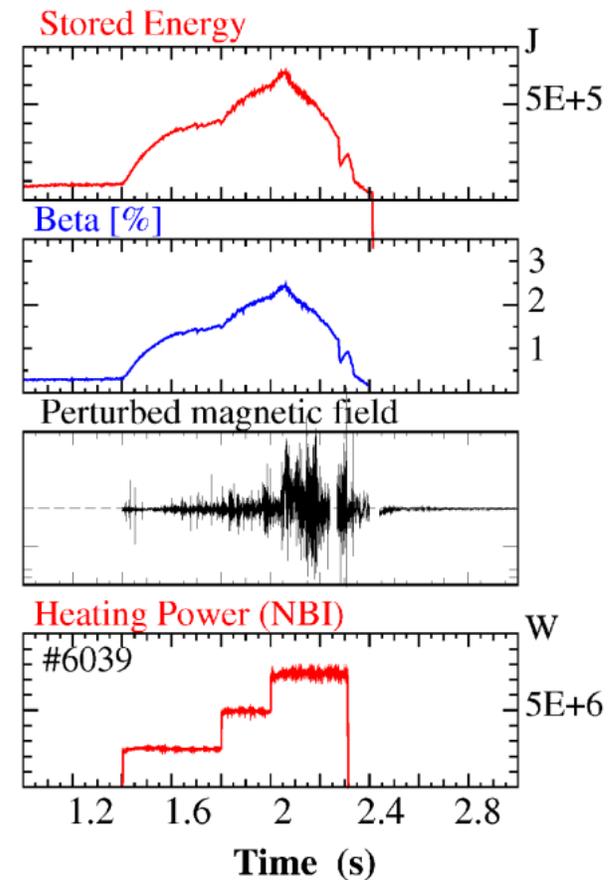
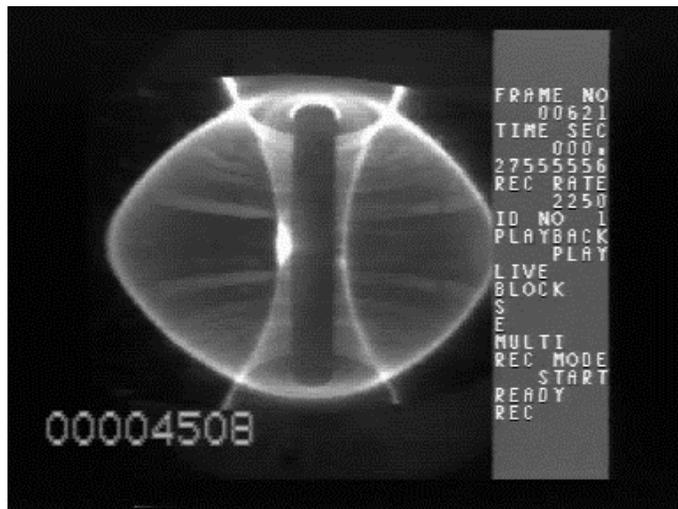
IR images (left): electrons of  $\sim 25 - 35$  MeV

Visible light images (right): electrons of  $> 60$  MeV

# Non-linear MHD Instabilities

- Major Disruption

- Several classes of “triggering” instabilities lead to this “final” ideal instability
  - Beta / pressure limits
  - Radiative limits
  - Vertical position instability (Vertical Displacement Event (VDE))

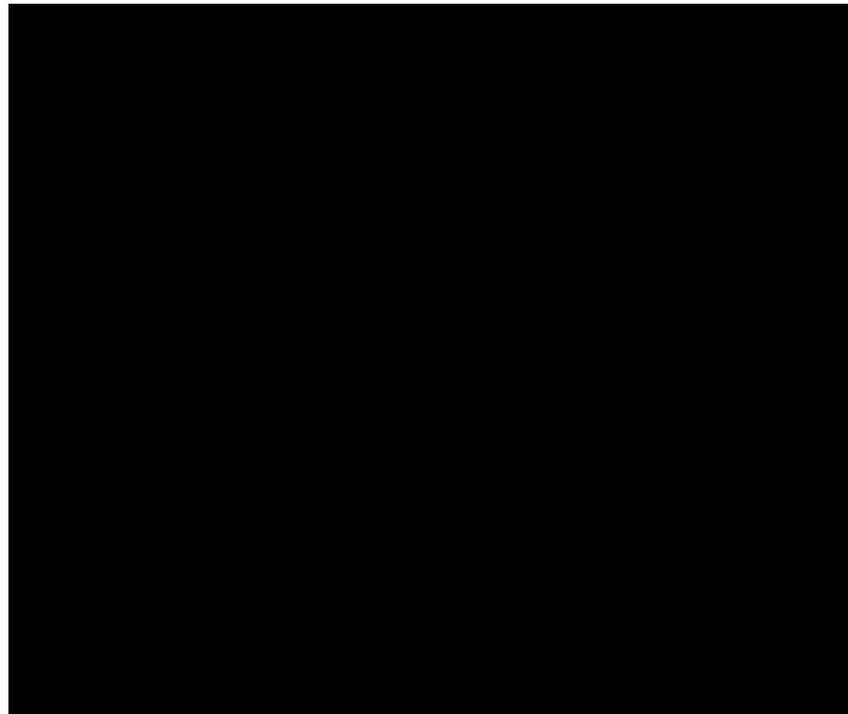


# Non-linear MHD Instabilities

- **Disruption Mitigation**

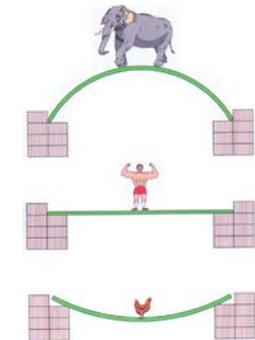
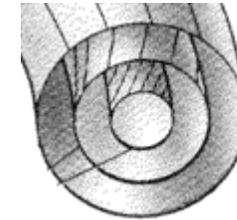
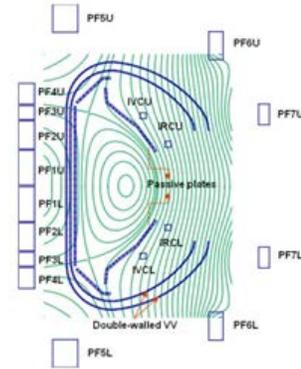
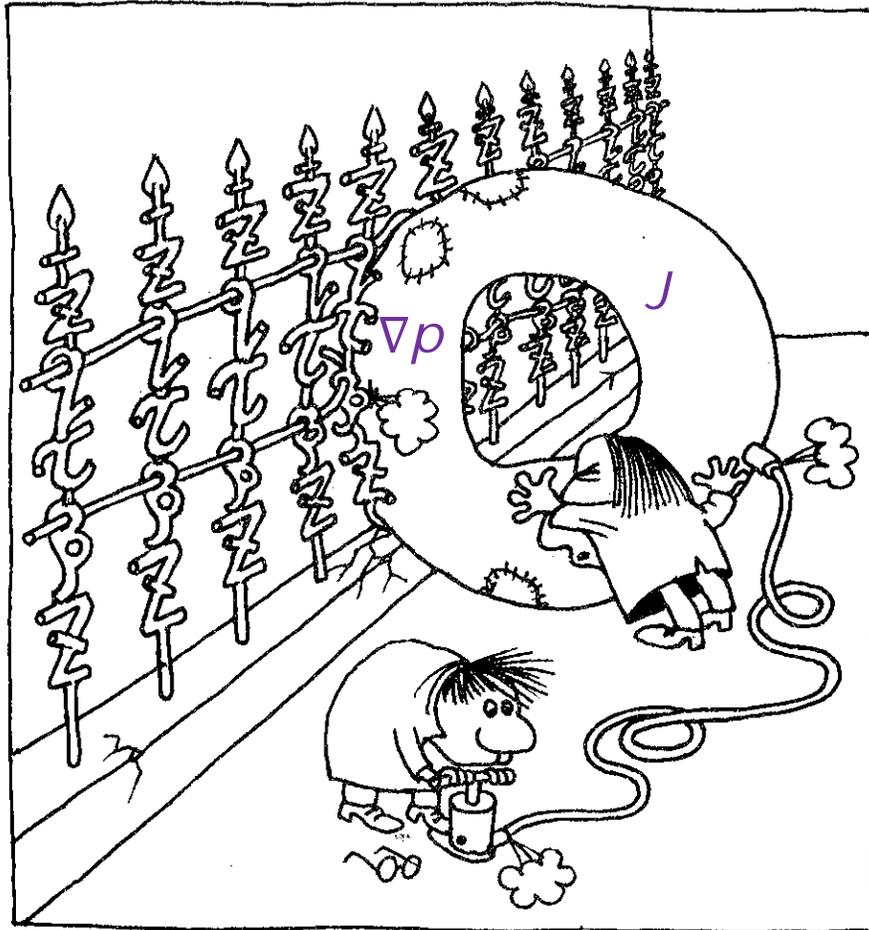
- Killer pellet injection: fast conversion of thermal energy to the radiation energy
- MGI (Massive Gas Injection): H, He, Ne, Ar, Kr, Xe, etc.
- RMP (Resonant Magnetic Perturbation) to reduce runaway electrons

Non-mitigated  
VDE



Neon gas jet  
injection  
triggered by  
control system

# Tokamak Instabilities and Their Control



- conducting wall
- magnetic shear
- minimum-**B** configuration
- profile optimisation
- dynamic stabilisation by feedback control
- ...

