Introduction to Nuclear Fusion

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Plasma wall interaction (PWI)

Plasma Confinement

Without magnetic field





 Minimise contact to material walls by magnetic field



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Fluxes to the Surface

Particle Fluxes

- Ions: diffusing fuel ions (H, D, T)

diffusing impurities (wall materials, residual gases in

chamber adsorbed gases in surface)

energetic ions from NBI, ICRH

- fusion alpha particles
- Electrons: diffusing plasma electrons runaway electrons
- Neutrals: recycling neutrals NBI neutrals i.e. shine-through
- Fusion neutrons

Fluxes to the Surface

• Energy Fluxes

- Particle energies
- Radiation energies
- Heat conduction

Photon Fluxes

- Bremsstrahlung radiation
- Cyclotron radiation
- Impurity line radiations

Plasma Radiation

- **Bremsstrahlung (Braking radiation)** \bullet
 - Process of radiation emission when a charged particle accelerates or decelerates
 - Contribution from ions can be neglected due to their heavier mass
 - $(m_p = 1836m_e)$ compared with that of electrons.
 - Mainly due to e-i collisions:

in e-e or i-i collisions, radiation fields exactly cancel.

- X-ray wavelength range ($\lambda \sim 10^{-9}$ m): readily escaping from a plasma



Plasma Radiation

- Cyclotron Radiation
 - Due to the centripetal acceleration of charged particles owing to the helical motion by magnetic field lines
 - contribution from ions can be neglected due to their heavier mass compared with that of electrons.
 - In the far infrared radiation spectrum ($\lambda = 10^{-3}$ - 10^{-4} m): partially re-absorbed in a plasma
 - The emitted radiation may be reflected from the surrounding wall in a magnetic confinement fusion device and thereby re-enter the plasma



$$r_{cyc}^{net} \approx 6.23 \times 10^{-20} n_e B^2 k T_e \psi \quad (W/m^{-3})$$

 ψ accounting for the complex processes of reflection and reabsorption of cyclotron radiation

http://www.astro.wisc.edu/~bank/index.html

Plasma Radiation

Line Radiation

- Due to radiative decay after electron collision excitation

$$A_0^{+Z} + e \rightarrow (A_0^{+Z})^* \rightarrow A_0^{+Z} + \gamma + E_{Zl}$$

$$P_L \approx 1.8 \times 10^{-38} n_Z n_e Z^4 \sqrt{T_e} ~(\text{W/m}^{-3})$$

Flux Excursions

- Transient Flux Excursions
 - Plasma instabilities can lead to transient heat load excursions.







Flux Excursions

Transient Flux Excursions

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Flux Excursions

Localised Flux Excursions



- Loss of fast particles can lead to excessive local heat loads
 - charge exchange neutrals by heating with NBI
 - orbit losses of fast ions
 - runaway electrons



Plasma wall interaction

Plasma-wall Interactions

• Importance of Plasma-Wall Interaction

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| KSTAR TV1 (t=-100m | is) | | |

KSTAR first H-mode plasma with good shape control (#4202)

Plasma-wall Interactions

- Physical/chemical interaction between plasma and (surrounding) surface
 - Reflection by backscattering
 - Adsorption and desorption
 - Physical sputtering: Erosion
 - Chemical sputtering
 - Vaporisation and melting
 - Blistering and flaking
 - Electron emission
 - Radiation damage and transmutation by
 - 14.1-MeV Neutron
 - Dust formation



http://www-rcp.ijs.si/mic/our_work/applications/fusion/fusion.php

Alteration of surface (wall erosion) and production of particle and photon fluxes \rightarrow Impurities

- Reflection by Backscattering
- Adsorption and Desorption
 - Residual cooler gases implanted inside wall \rightarrow release of gases

• Physical Sputtering – Erosion

- Ejection of surface atom from (low-temperature) wall as a result of collision cascade in the lattice atoms by particles when acquired energy > surface binding energy
- Threshold exists in incident energy to produce sputtering





- Physical Sputtering Erosion
 - For beryllium and tungsten, theoretical and experimental yields agree very well.
 - Carbon shows additional erosion with only weak dependency on impact energy
 - \rightarrow Chemical erosion



Chemical Sputtering

- Chemical reaction of incident projectiles with target atoms
- Formation of a volatile chemical compound leaving the solid: occurs only for certain target-projectile combinations



High Field Side (HFS) inner wall



antenna protection/outer wall



Vaporisation and Melting

- disruptive instabilities \rightarrow thermal shock on the wall

 \rightarrow spalling, cracking, melting, evaporation

• Blistering and Flaking

- Blistering: gas bubble in $\sim \mu$ -thick surface layer (insoluble, He)
- Flaking: blister rupture by lateral stress and surface layer breaking



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Electron Emission

- Photoelectric, thermionic, X-ray, secondary

Radiation Damage and Transmutation by 14.1-MeV Neutron

- Knock-on collision \rightarrow interstitial, spikes, voids, displacements, ...
- Neutron capture reactions: (n,p), $(n,a) \rightarrow$ production of p, a in the first wall \rightarrow swelling, radiation damage of wall, diffusing back to plasma

Dust Formation

- ITER definition: solid particles/debris of size about 10 nm-100 μm
- Consequence of PWI/volume polymerisation in edge plasma
- Safety and operational issue (limit)



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Mobilised "dusts" (Tore Supra)





Nanoparticles/metal droplet (Tore Supra/JET/KSTAR)

Suk-Ho Hong, "Current plasma-wall interaction activities in KSTAR", SNU Seminar, Seoul, 5 February, 2010

How to control impurity influx?

Maximal Permissible Impurity Concentration



At 10 keV

- Low Z (~10): $(n_Z/n_{DT})_{max} \sim 10\%$ (Ne)
- Medium Z (~26): $(n_Z/n_{DT})_{max}$ ~0.5% (Fe)
- High Z (~74): $(n_Z/n_{DT})_{max}$ ~0.01% (W)



Maximal concentrations for sustained ignited plasma

• Maintain



- Plasma Boundary Region Control
 - Limiter: a material structure protruding from the main wall used to intercept particles at the plasma edge particularly to stop runaway electrons
 - from damaging the vacuum vessel and to shadow in-vessel components from the plasma edge (limiting/defining the plasma size)

 - \rightarrow Reflecting neutrals \rightarrow Pumping out

- Plasma Boundary Region Control
 - Limiter



Last Closed Flux Surfaces (LCFS):

The magnetic surface that touches the innermost part of the limiter

Scrape-off Layer (SOL):

The plasma region located in the limiter shadow

i.e. between the LCFS and the vessel wall

- Plasma Boundary Region Control
 - Limiter problems
 - High heat load and sputtering rate on limiter
 - \rightarrow Impurities \rightarrow Low-Z coating of limiters (C or Be on W)



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Plasma Boundary Region Control

Proposed by L. Spitzer for stellarators

 Divertor: Bending outer magnetic fields away from plasma by means of auxiliary magnetic coils → Removing outer layer of plasma to external chamber → Cooling → Neutralising → Pumping away



The magnetic boundary between confined plasma and edge/divertor plasma is called **separatrix = LCFS**

- Plasma Boundary Region Control
 - Divertor functions

Reduction of 1st wall bombardment (unloading) Reduction of impurity flow into plasma (shielding for impurity control) Exhaust plasma particles and power and removal of He ash

- Divertor problems
 - Complex coil systems High cost Difficult maintenance

The divertor in ASDEX Upgrade



- Plasma Boundary Region Control
 - Divertor







http://nuclearfusionwilliaco.weebly.com/the-tokamak.html

http://energyphysics.wikispaces.com/Tokamak+Thrust+Engine https://www.iter.org/fr/album/media/7%20-%20technical